



Status of He-cooled Divertor Design

Presented by Thomas Ihli *

Contributors: A.R. Raffray, and
The ARIES Team

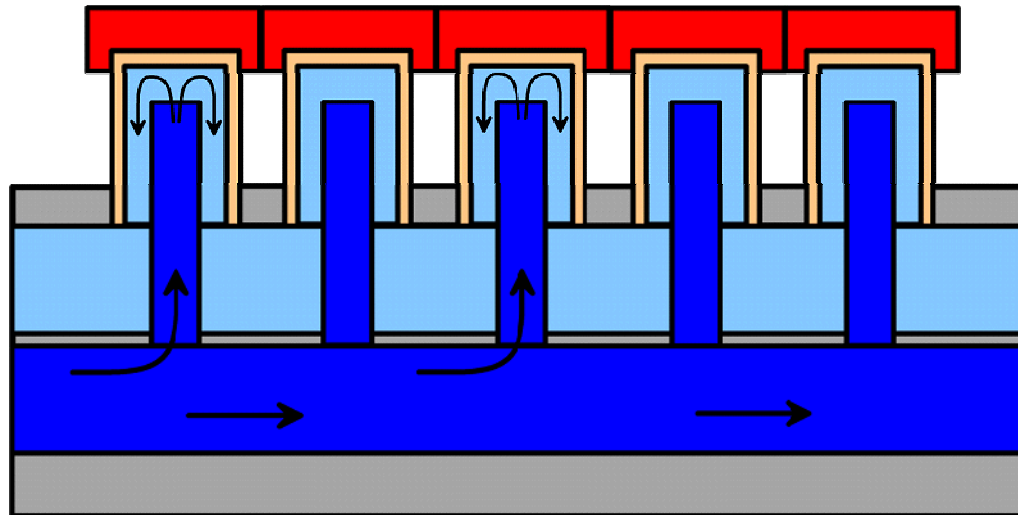
ARIES Meeting
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* Forschungszentrum Karlsruhe, currently exchange visitor to UCSD & The ARIES Team



Basic divertor concept

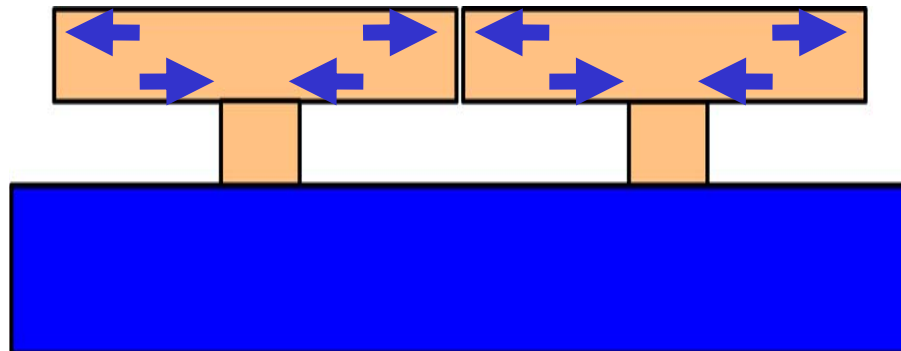
Caps in parallel



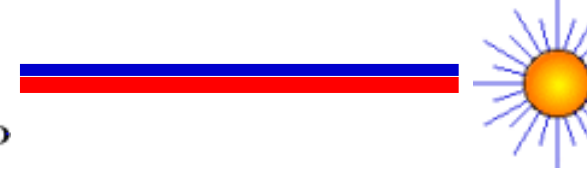
heat exchange part:
tungsten alloy

target main structure:
ODS ferritic steel

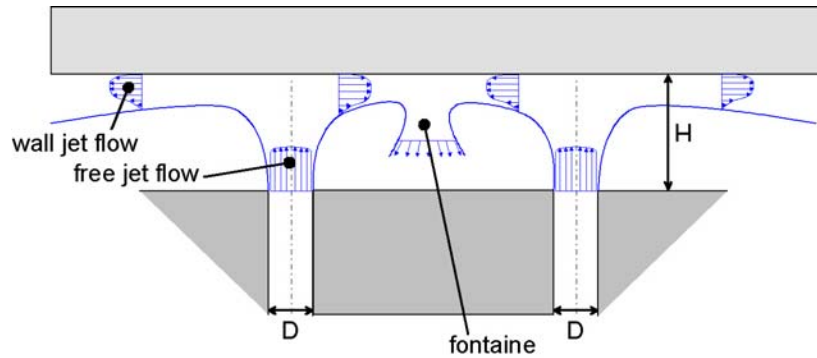
T-Tubes in parallel



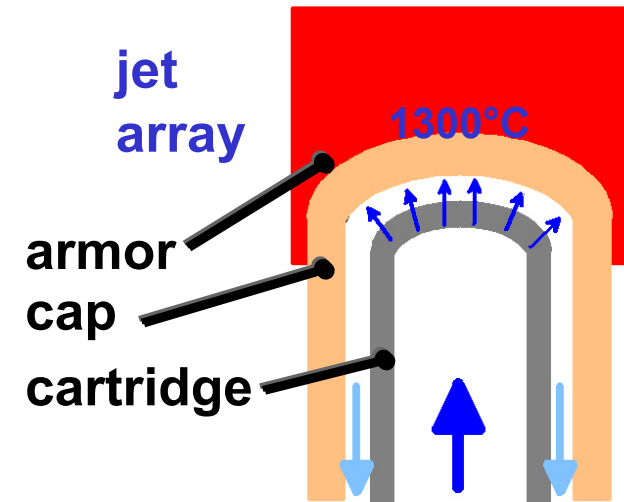
- + reduced number of single parts
- + single parts can be tested before assembly
- + simplified manifold possible (more space)



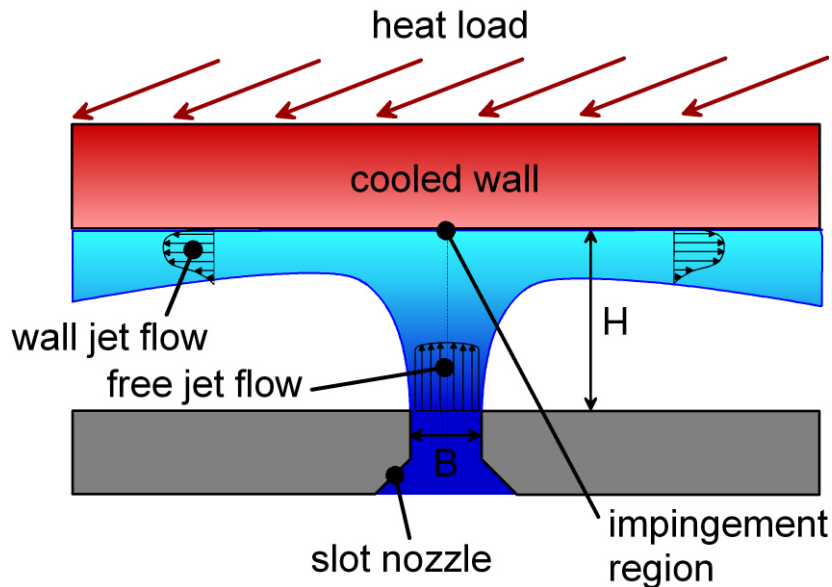
Cooling concept



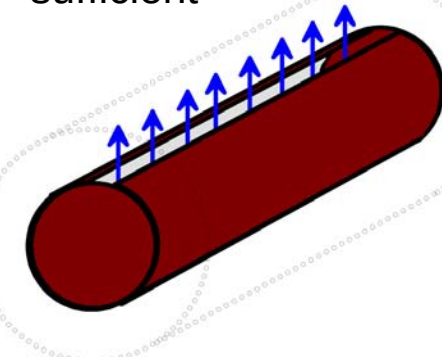
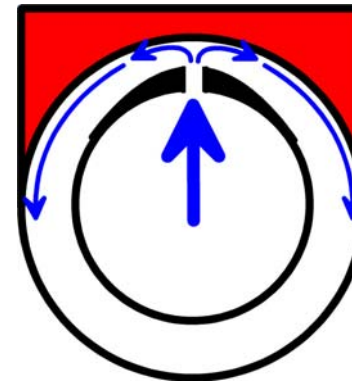
Multijet



Slot Jet

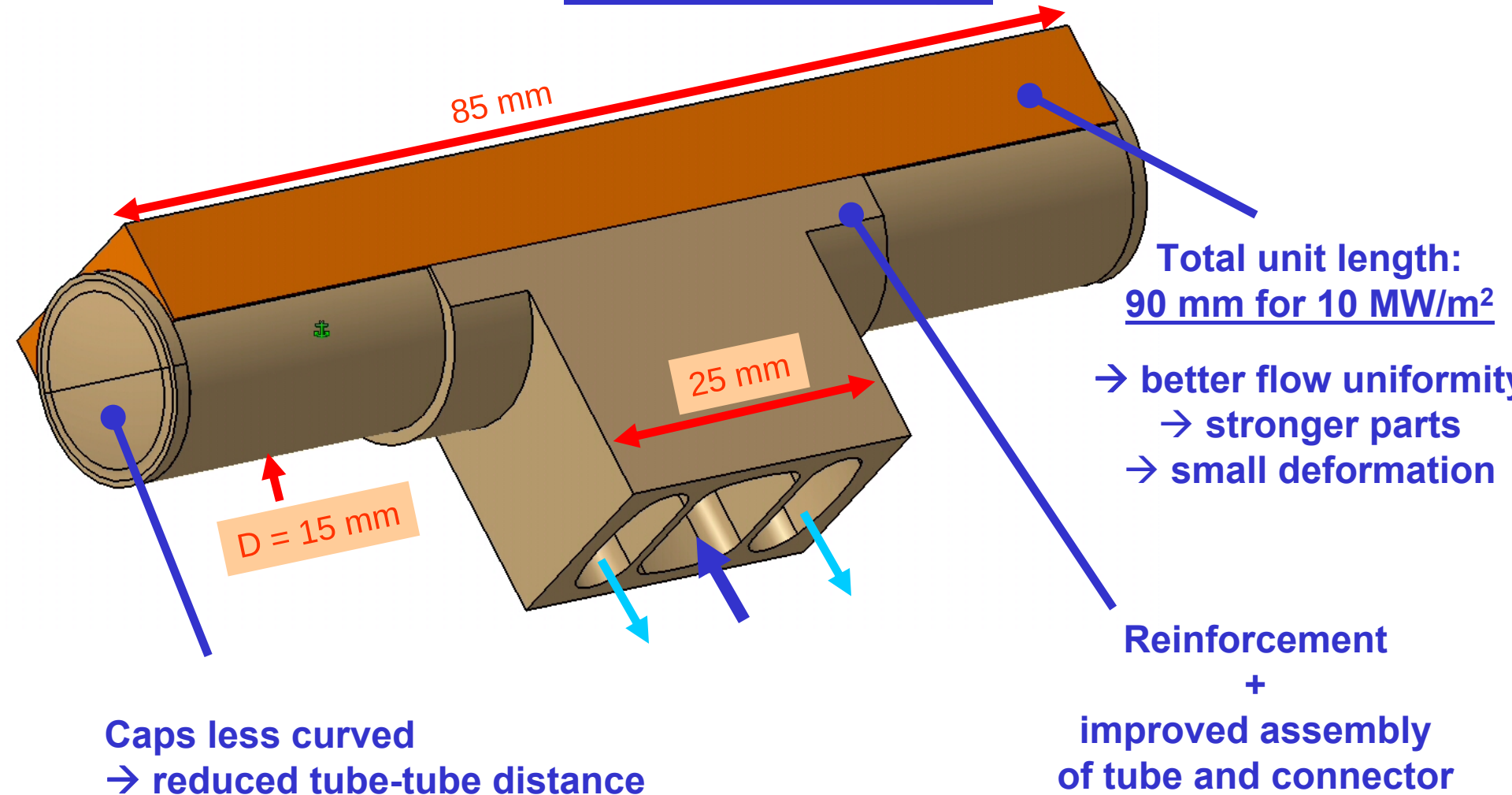


+ larger cooling area
→ lower averaged
heat transfer coefficient
sufficient



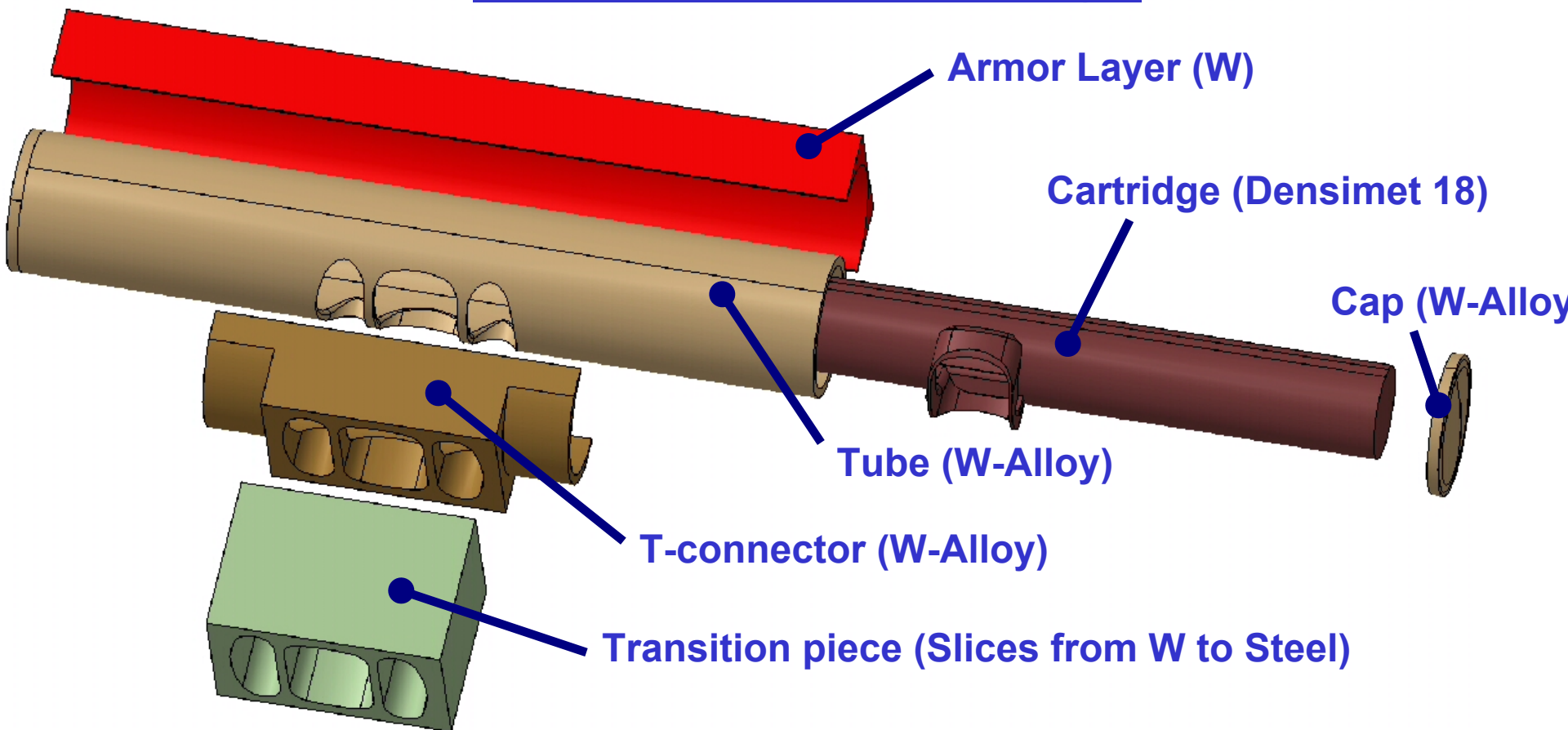


Refined Design



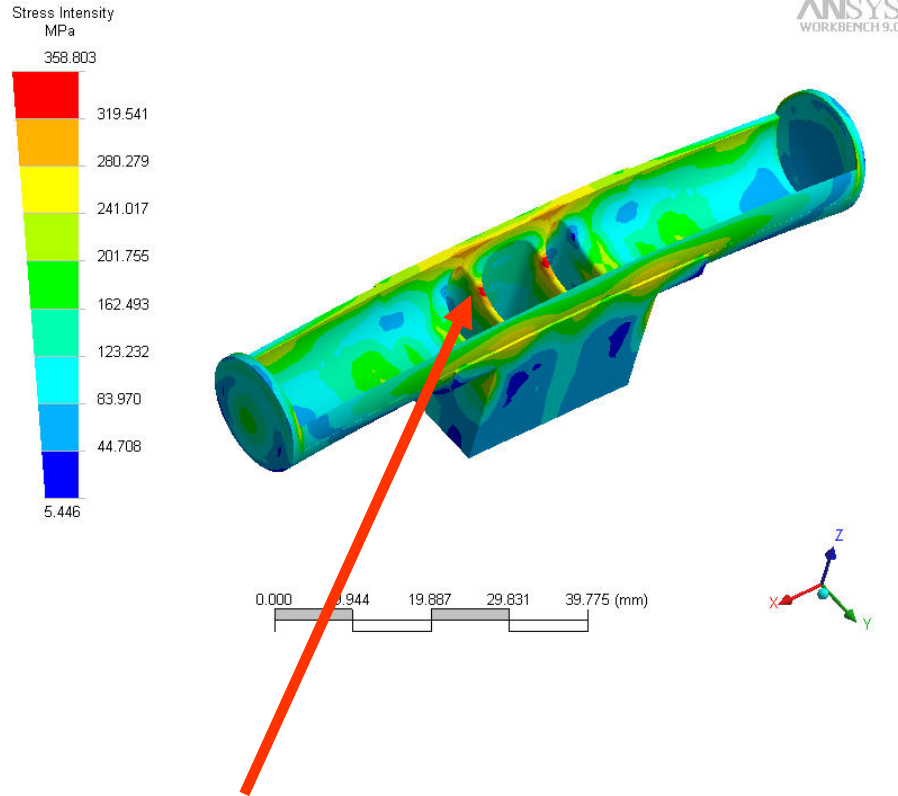


Parts of T-Tube Design



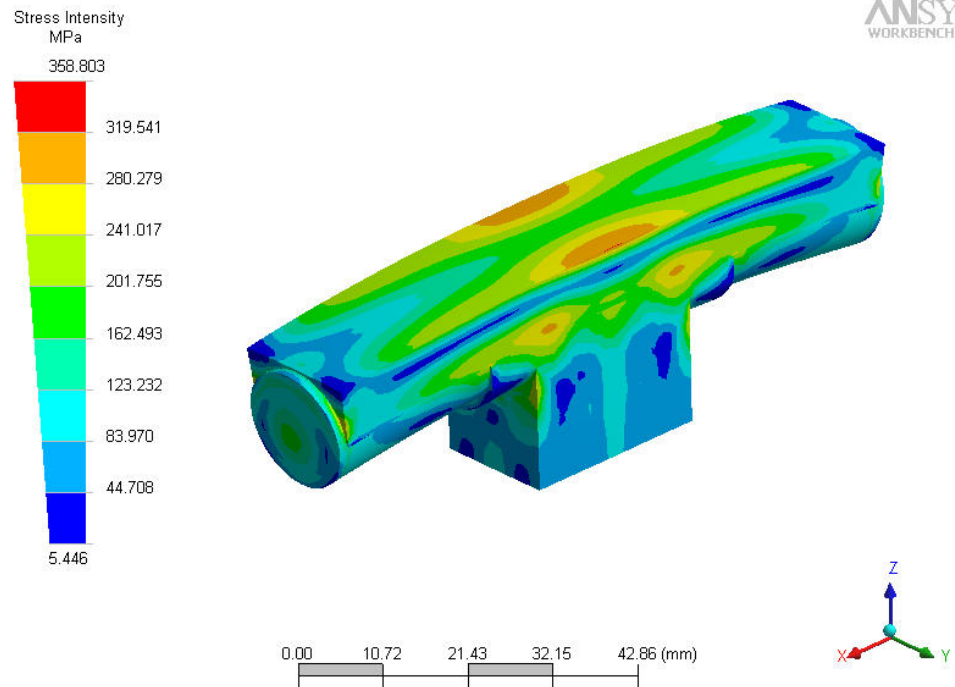


Stress Intensities for basic geometry



**acceptable total stress
at flow channels**

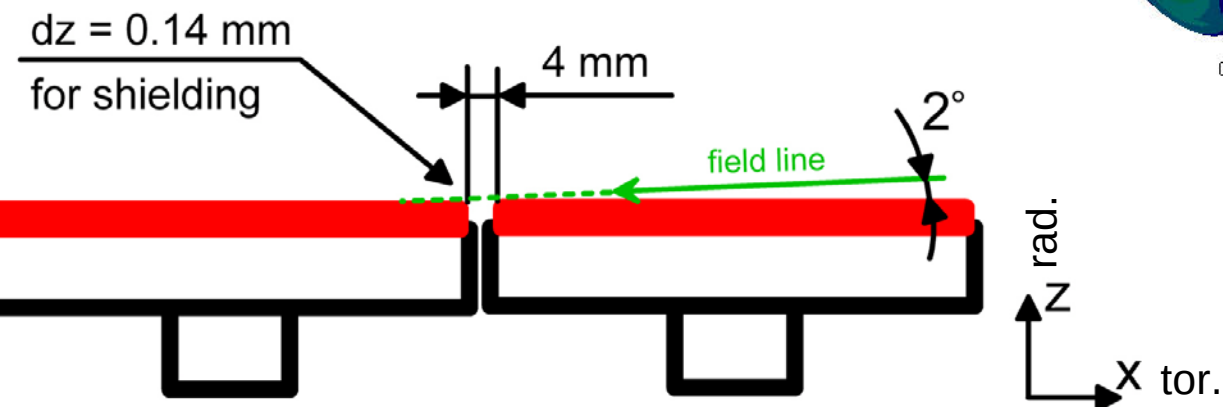
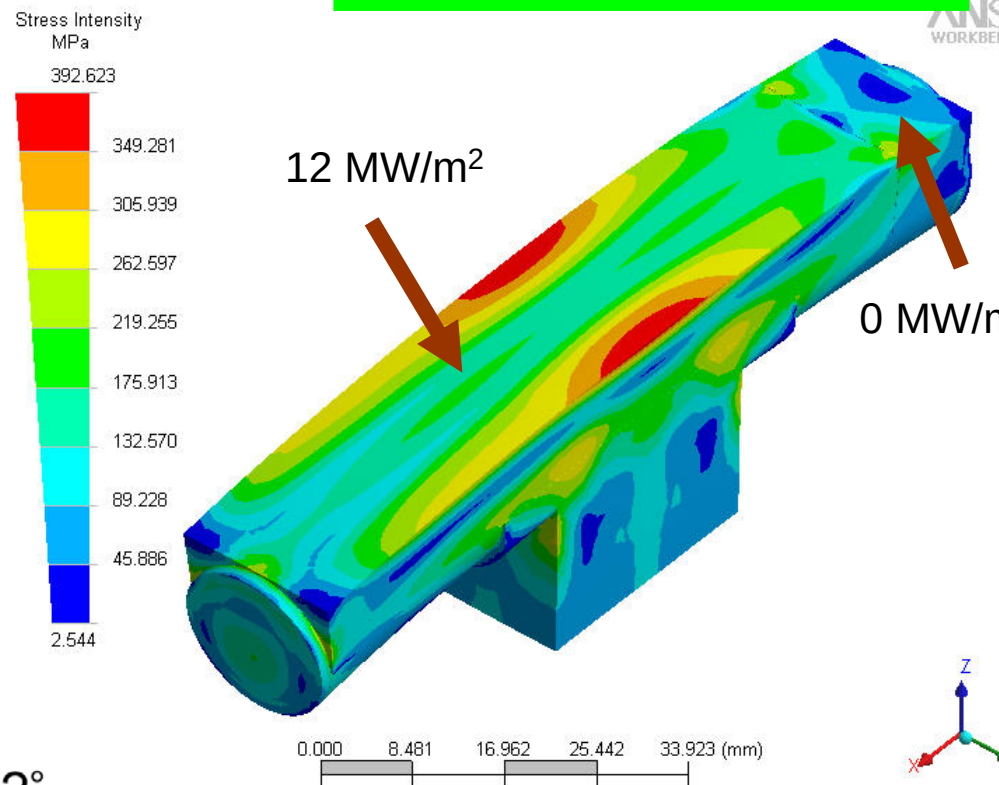
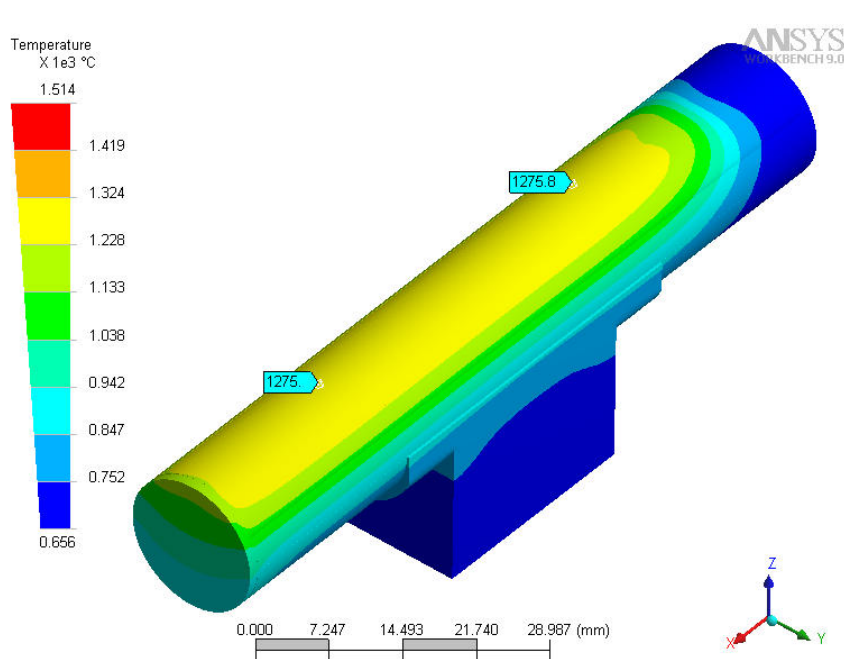
**maximum of stress intensities
(~ 360 MPa) within design limits**





Shielding

asymmetric heat load

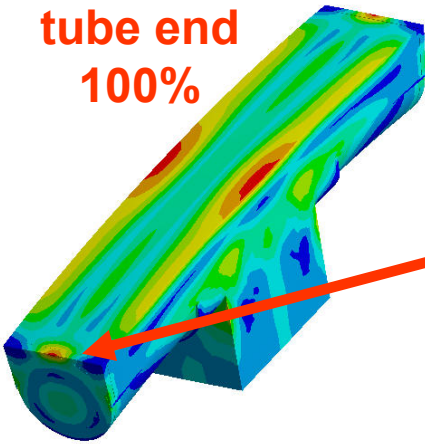




Heat load on caps (by radiation)

Heat load at
tube end
100%

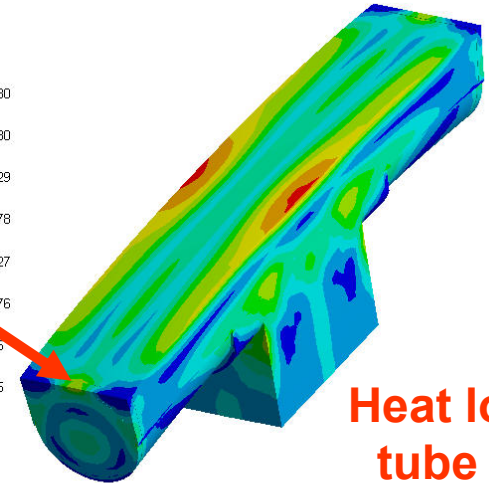
Stress Intensity
MPa
370.382
329.944
289.505
249.066
208.627
168.189
127.750
87.311
46.872
6.434



ANSYS
WORKBENCH 9.0

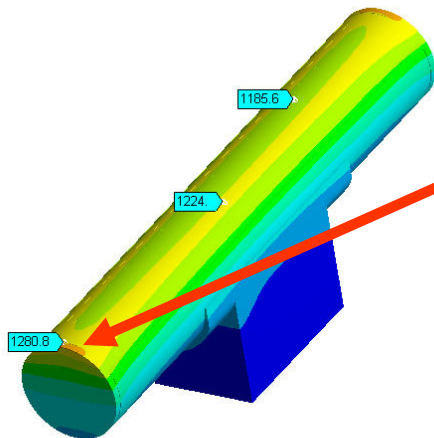
stress intensities
at tube end o.k.

Stress Intensity
MPa
366.981
327.130
287.280
247.429
207.578
167.727
127.876
88.025
48.175
8.324



Heat load at
tube end
50%

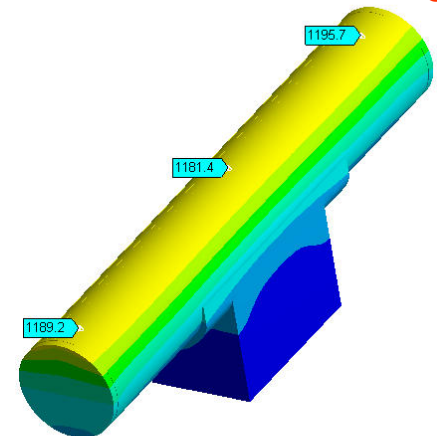
Temperature
X 1e3 °C
1.485
1.395
1.305
1.216
1.126
1.037
0.947
0.857
0.768
0.678

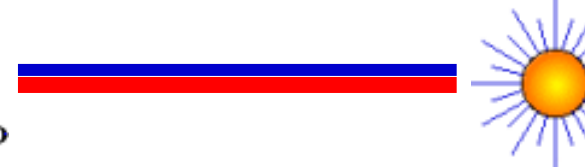


ANSYS
WORKBENCH 9.0

Temperature peak
(for more than 50 %
load on tube ends)

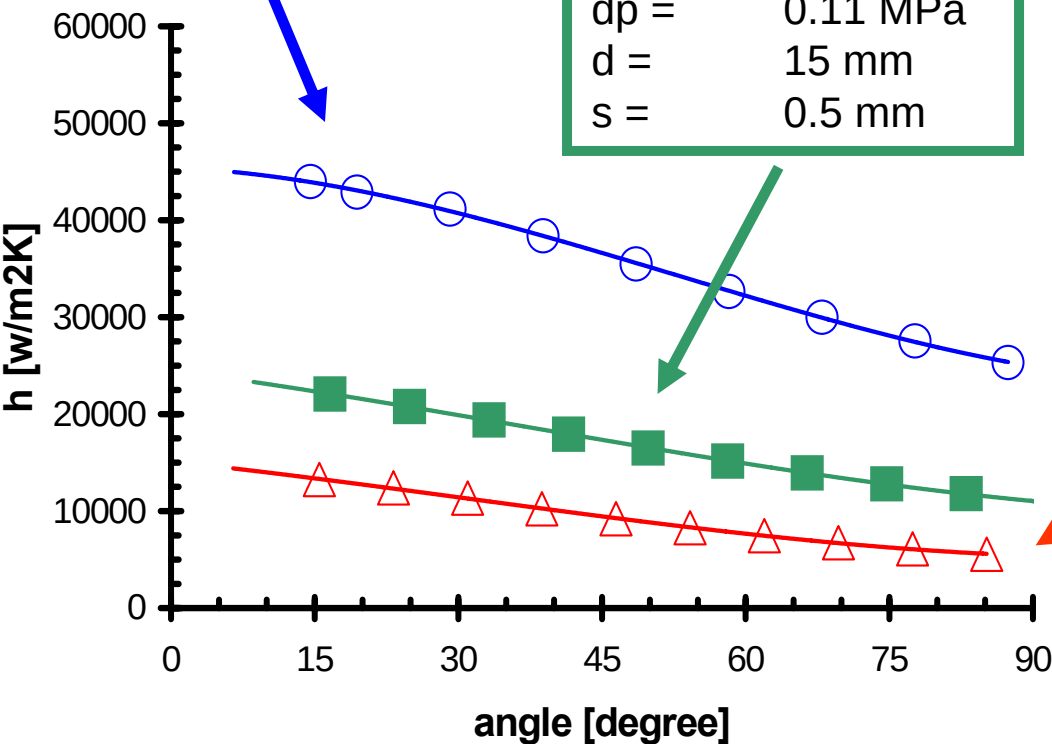
Temperature
X 1e3 °C
1.424
1.341
1.258
1.175
1.092
1.010
0.927
0.844
0.761
0.678



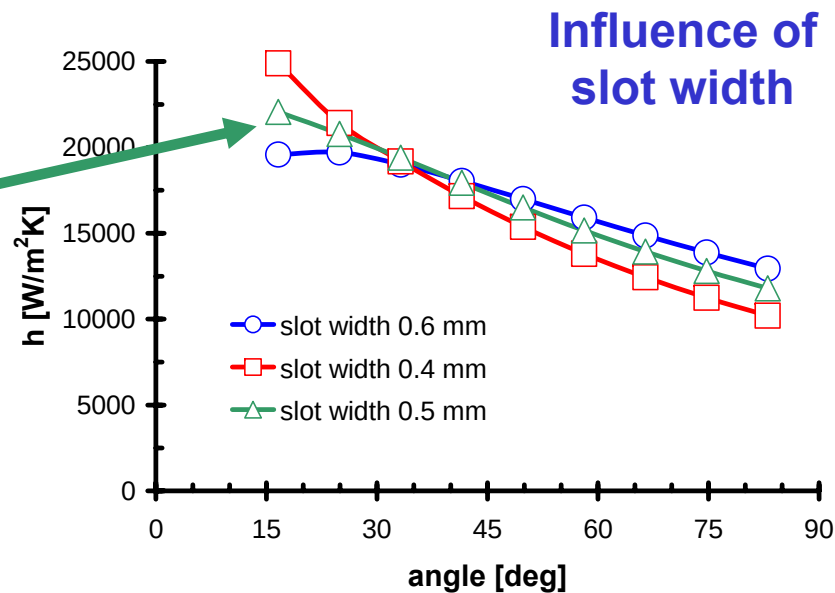


Heat transfer coefficient

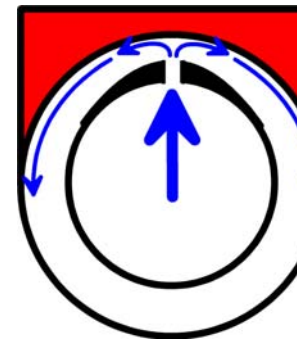
$q' = 20 \text{ MW/m}^2$
 $G/A = 48 \text{ kg/m}^2$
 $dp = 0.28 \text{ MPa}$
 $d = 7.5 \text{ mm}$
 $s = 0.25 \text{ mm}$



$q' = 10 \text{ MW/m}^2$
 $G/A = 24 \text{ kg/m}^2$
 $dp = 0.11 \text{ MPa}$
 $d = 15 \text{ mm}$
 $s = 0.5 \text{ mm}$



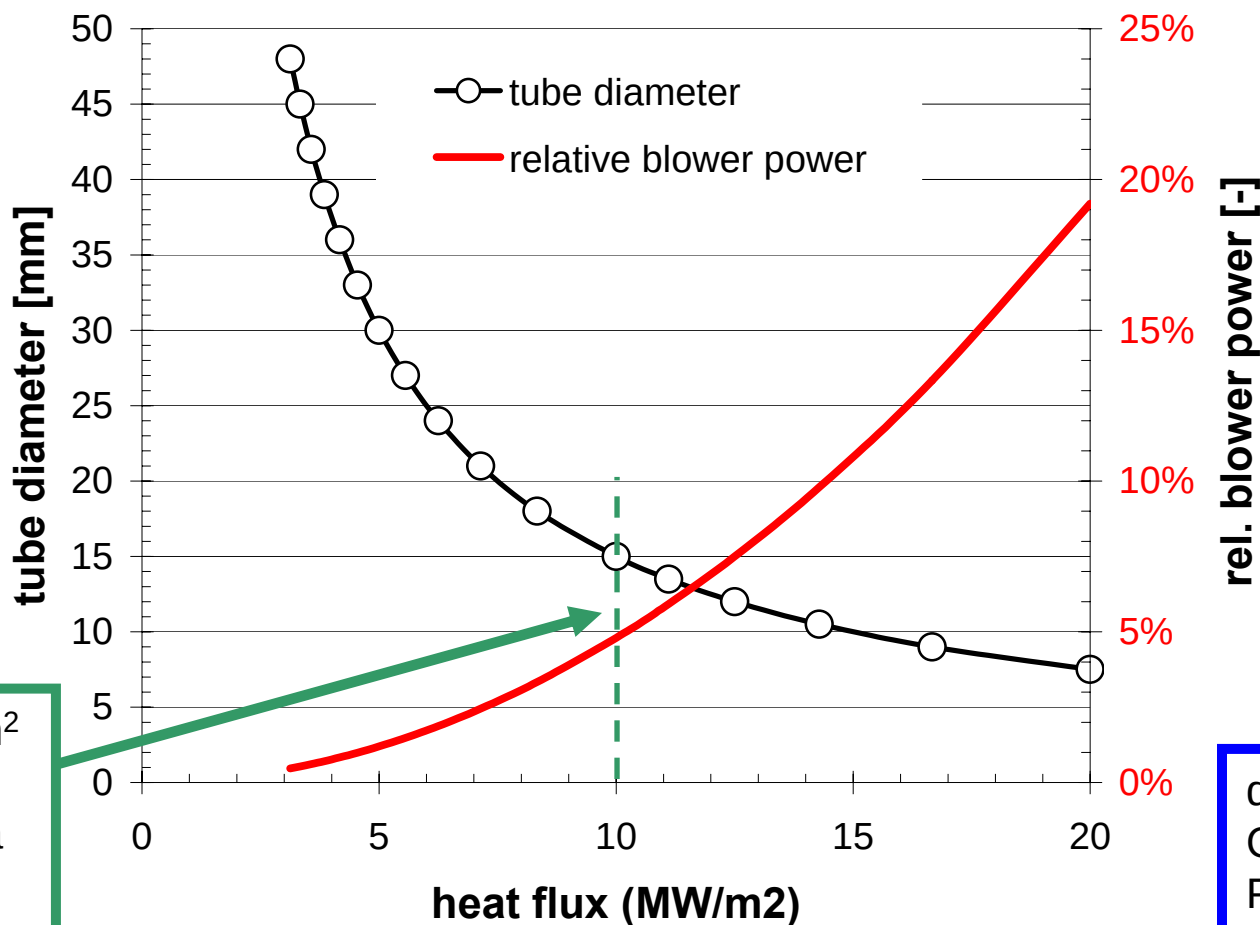
$q' = 5 \text{ MW/m}^2$
 $G/A = 12 \text{ kg/m}^2$
 $dp = 0.027 \text{ MPa}$
 $d = 30 \text{ mm}$
 $s = 0.8 \text{ mm}$





Adjustment to various loads by scaling

stress
intensities
~ const.

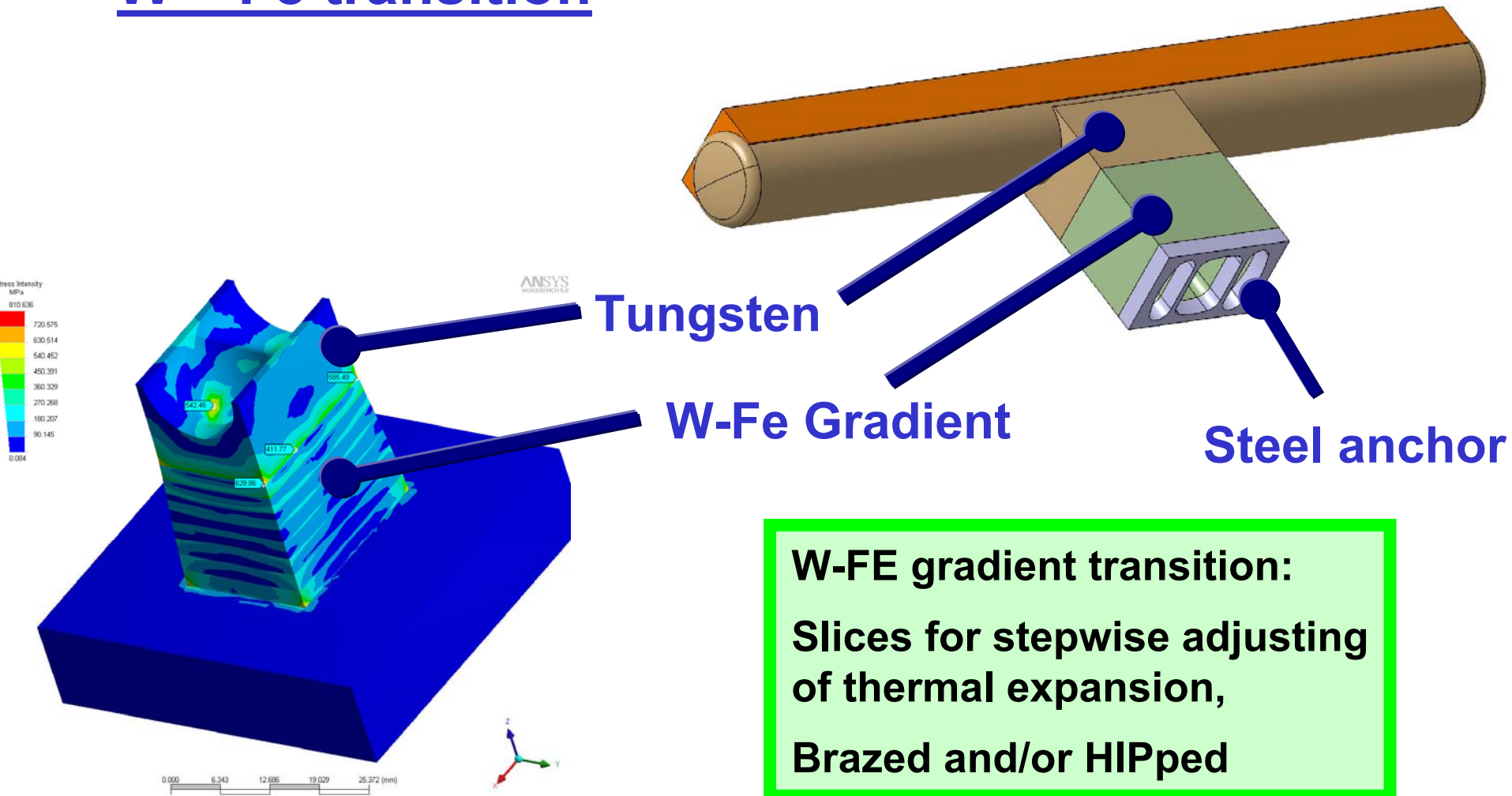


$q' = 10 \text{ MW/m}^2$
 $G/A = 24 \text{ kg/m}^2$
 $\Delta p = 0.11 \text{ MPa}$
 $d = 15 \text{ mm}$
 $s = 0.5 \text{ mm}$

$d, b, l \sim 1/q'$
 $G/A \sim q'$
 $P/Q' \sim q'^2$

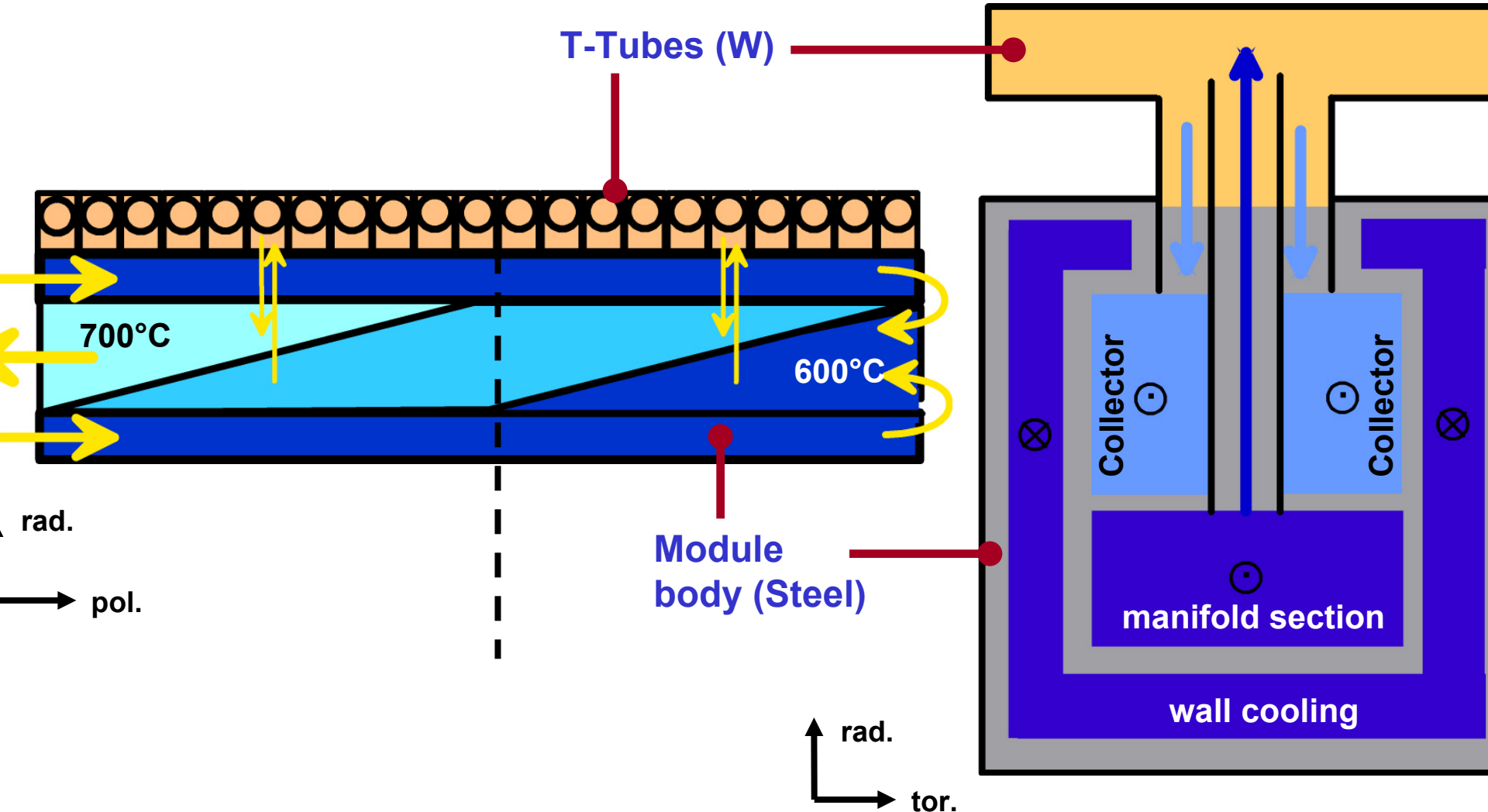


W – Fe transition



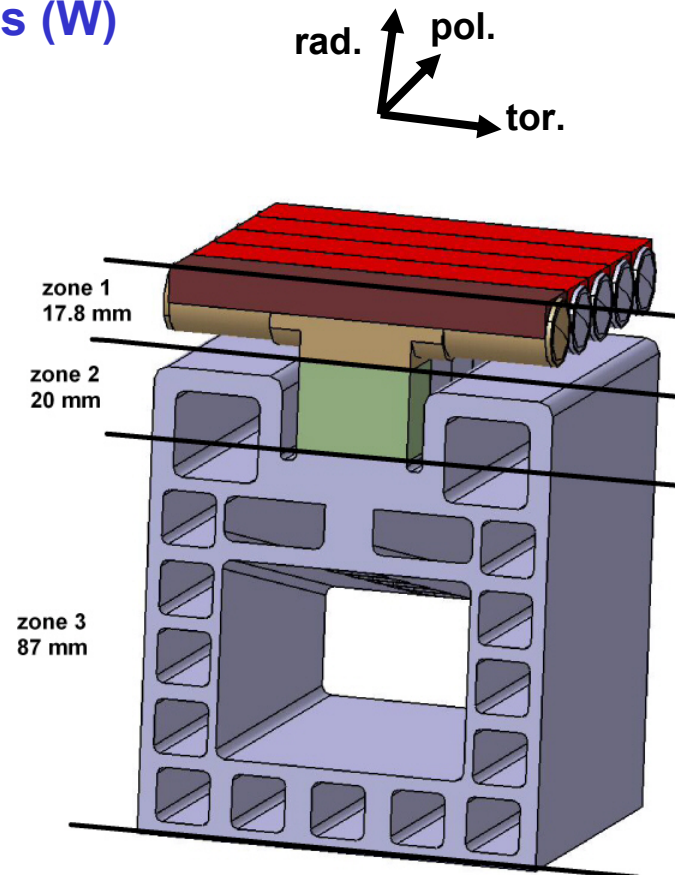
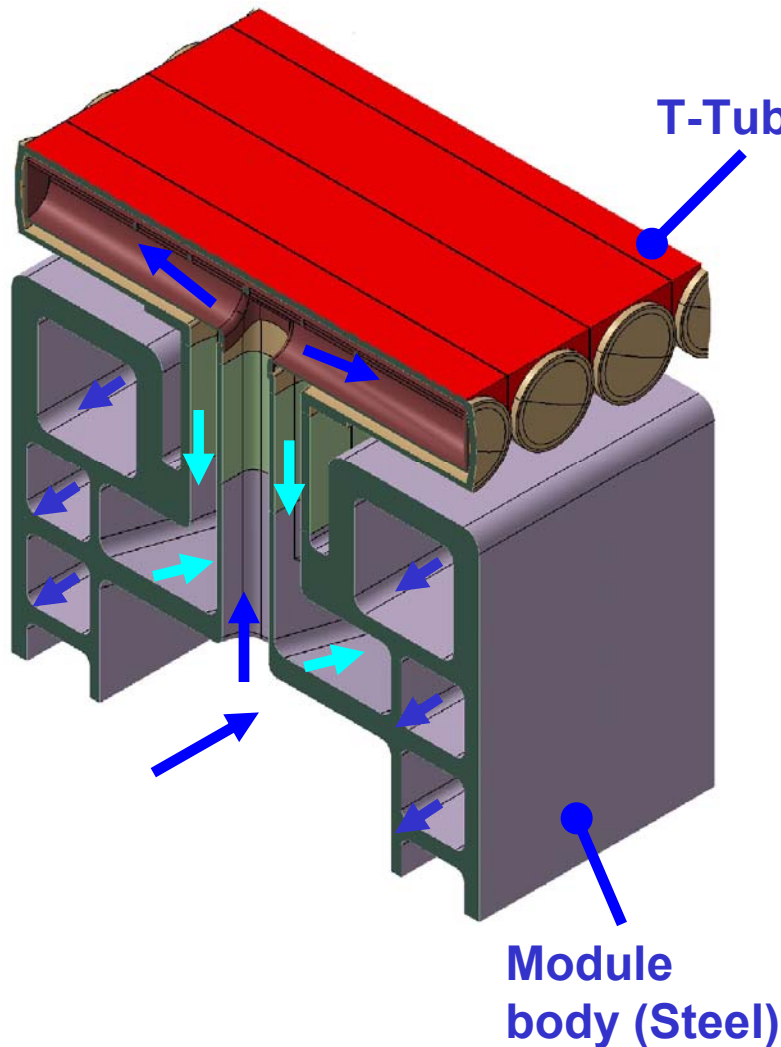


Schematic Divertor Module Design





Divertor Module Design



zone1: Tungsten parts, h = 17.8 mm

	V m3	%
sum W	9.540E-06	37.69%
Helium	1.07E-05	42.27%
space	5.07E-06	20.04%
total	2.53E-05	

zone2: W-FS-Gradient zone, h = 20 mm

	V m3	%
sum W	1.919E-06	6.75%
sum FS	9.909E-06	34.84%
Helium	1.05E-05	36.96%
space	6.10E-06	21.45%
total	2.84E-05	

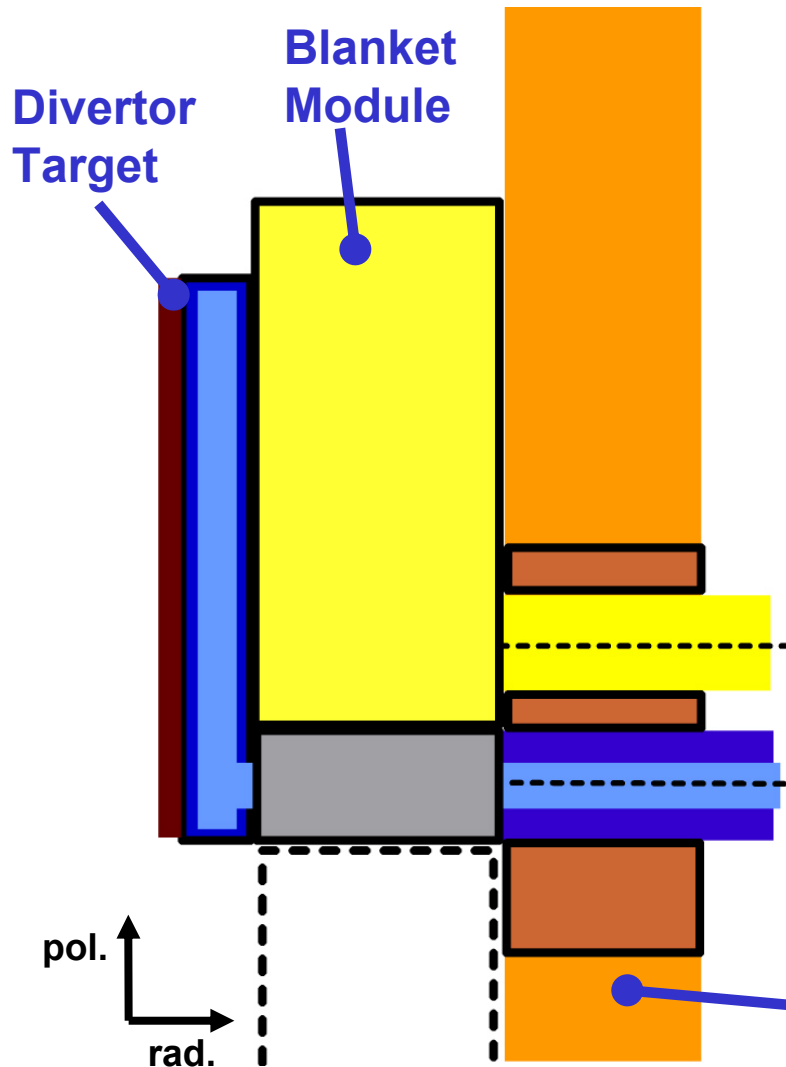
zone3: Manifolds, h = 87 mm

	V m3	%
sum FS	4.000E-05	32.34%
Helium	8.35E-05	67.50%
space	2.08E-07	0.17%
total	1.24E-04	



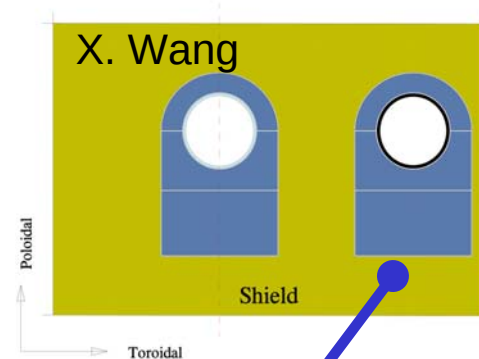
Divertor Integration:

Independent Blanket/Divertor
(if remaining breeding zone is still sufficient)

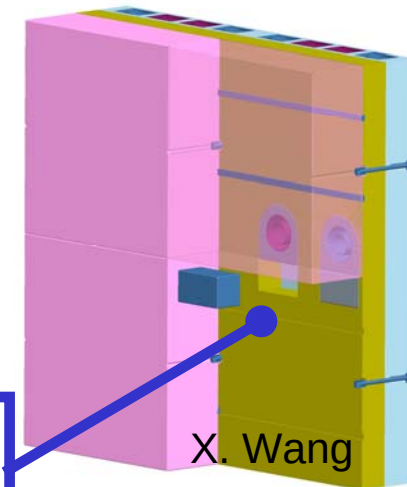


A1) Divertor Manifold box below Blanket module

- + Access to blanket pipes from below
→ same cutting/welding scheme for divertor
- + Simple shape for blanket
- Large cooled divertor manifold necessary
- reduced breeding zone
- divertor has to be disassembled from blanket



HT Shield with cut-outs
for front access maintenance

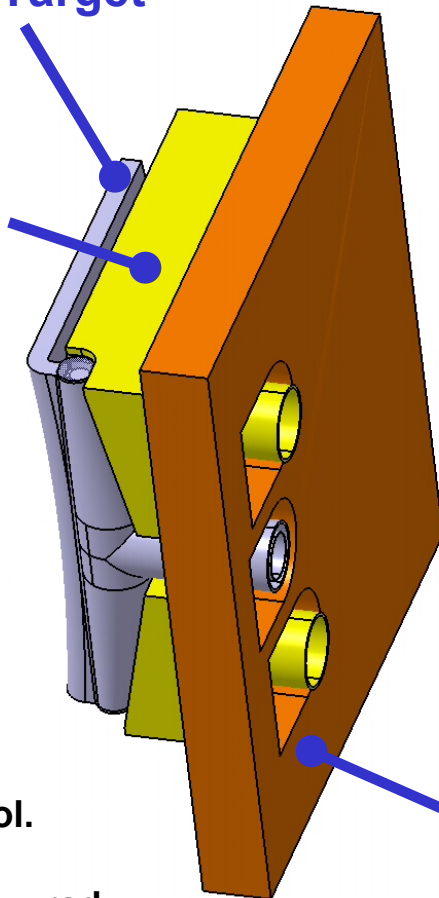




Divertor Integration

Divertor Target

Blanket
Module



Independent Blanket/Divertor
(if remaining breeding zone still sufficient)

A2) Divertor Manifold fits into cut-out in Blanket module

- + Divertor may be removed with Blanket
- + Access to blanket pipes from below
- + Smaller divertor manifold

- Very complicated shape for blanket
- Cooled divertor manifold necessary
- Reduced shielding in manifold/tube area

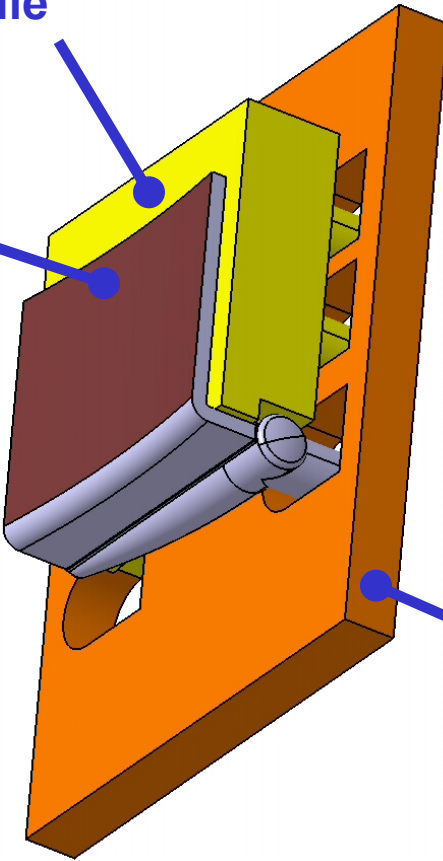
HT Shield with cut-outs
for front access maintenance



Divertor Integration Options

Blanket
Module

Divertor
Target

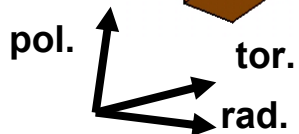


OPTION B:

Divertor Manifold fits into cut-out in Blanket module

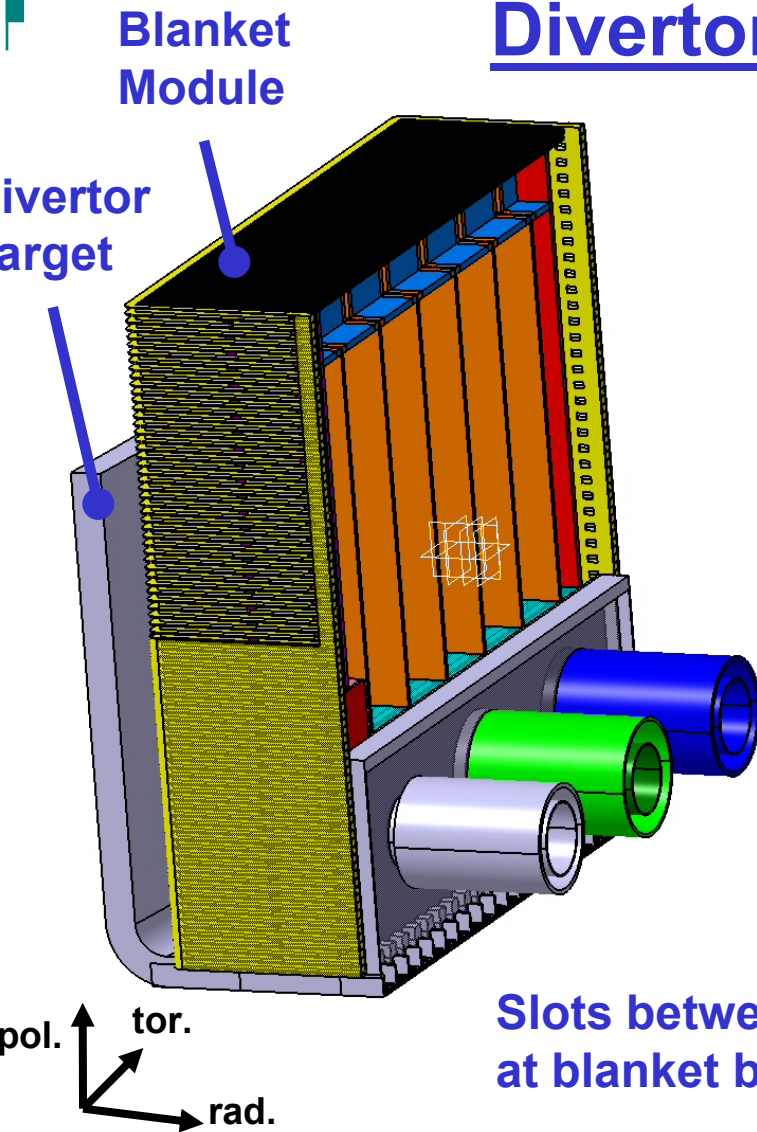
- + Independent Blanket/Divertor
- + Less neutron heating for divertor manifold
- Complicated shape for blanket
- Access of blanket pipes limited (tor.)
- Divertor cannot be removed without the Blanket

**HT Shield with cut-outs
for front access maintenance**



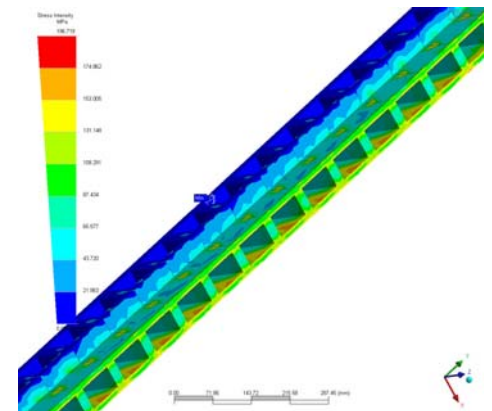


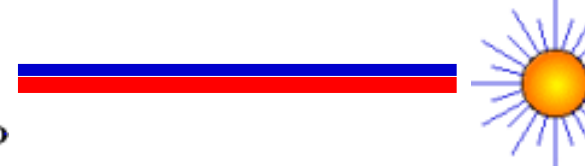
Divertor Integration Options



OPTION C: Divertor Manifold in Blanket box

- + Lower neutron heating for divertor manifold
- + No additional cooling for divertor manifold
- + Thin manifold as included in blanket box
- + Uncomplicated shape for blanket
- + Better shielding than option A & B
- + Access to blanket pipes from below
- Fix point at blanket back side
- Divertor part of blanket

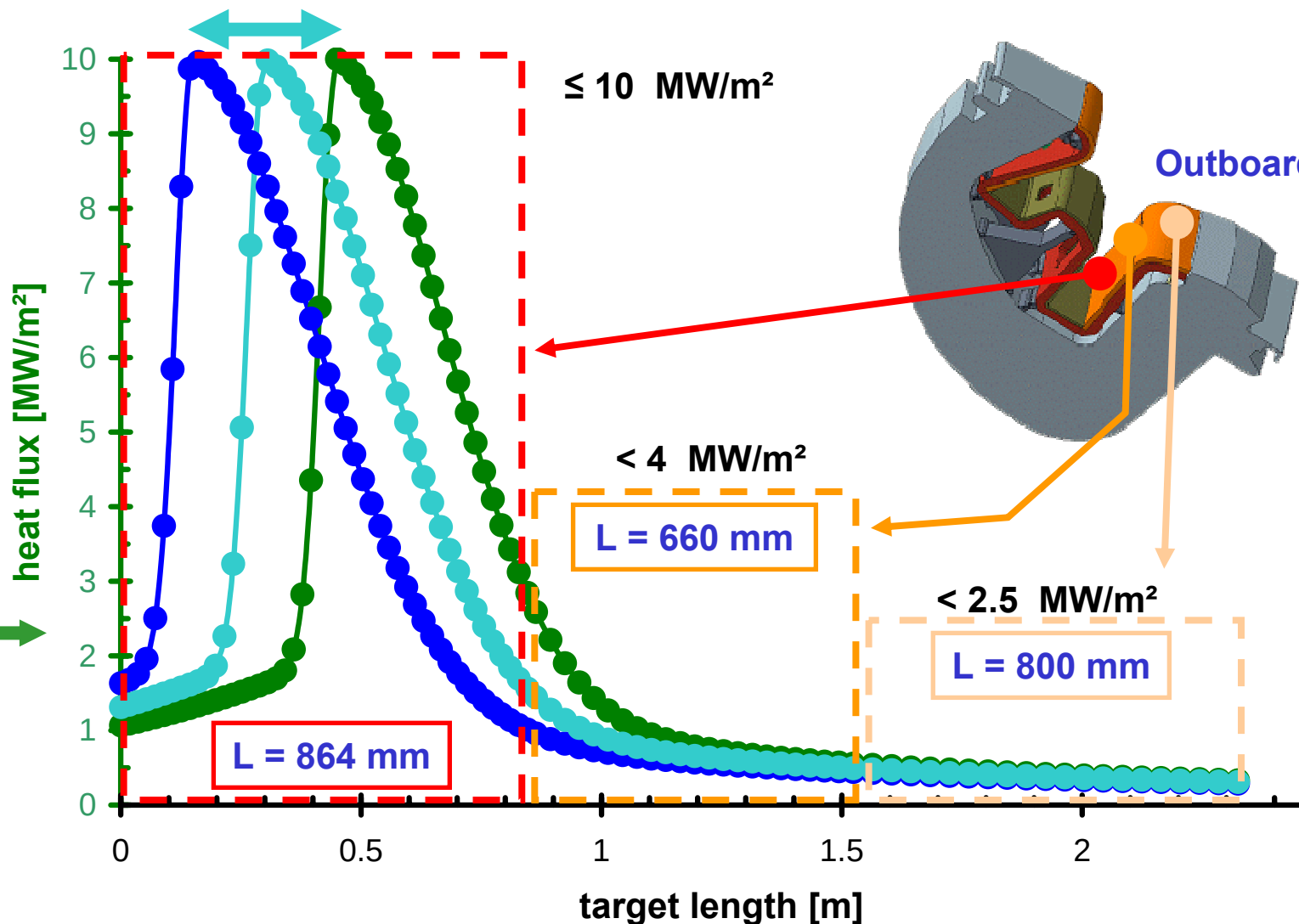




TOKAMAK Outboard target: cooling zones

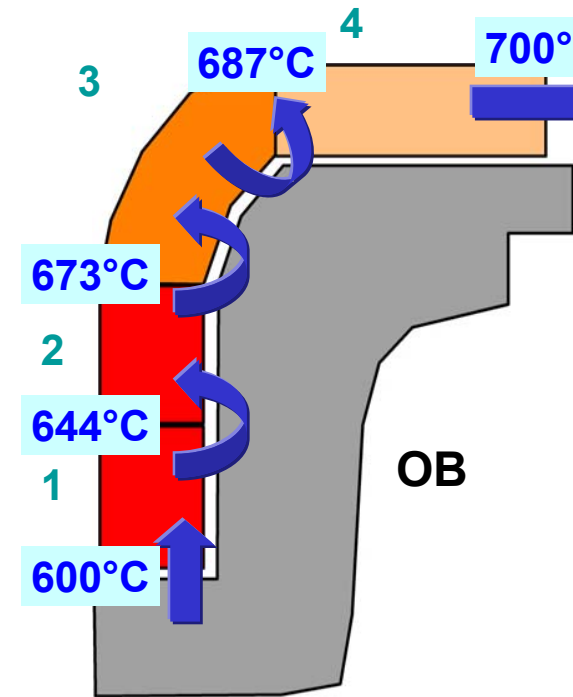
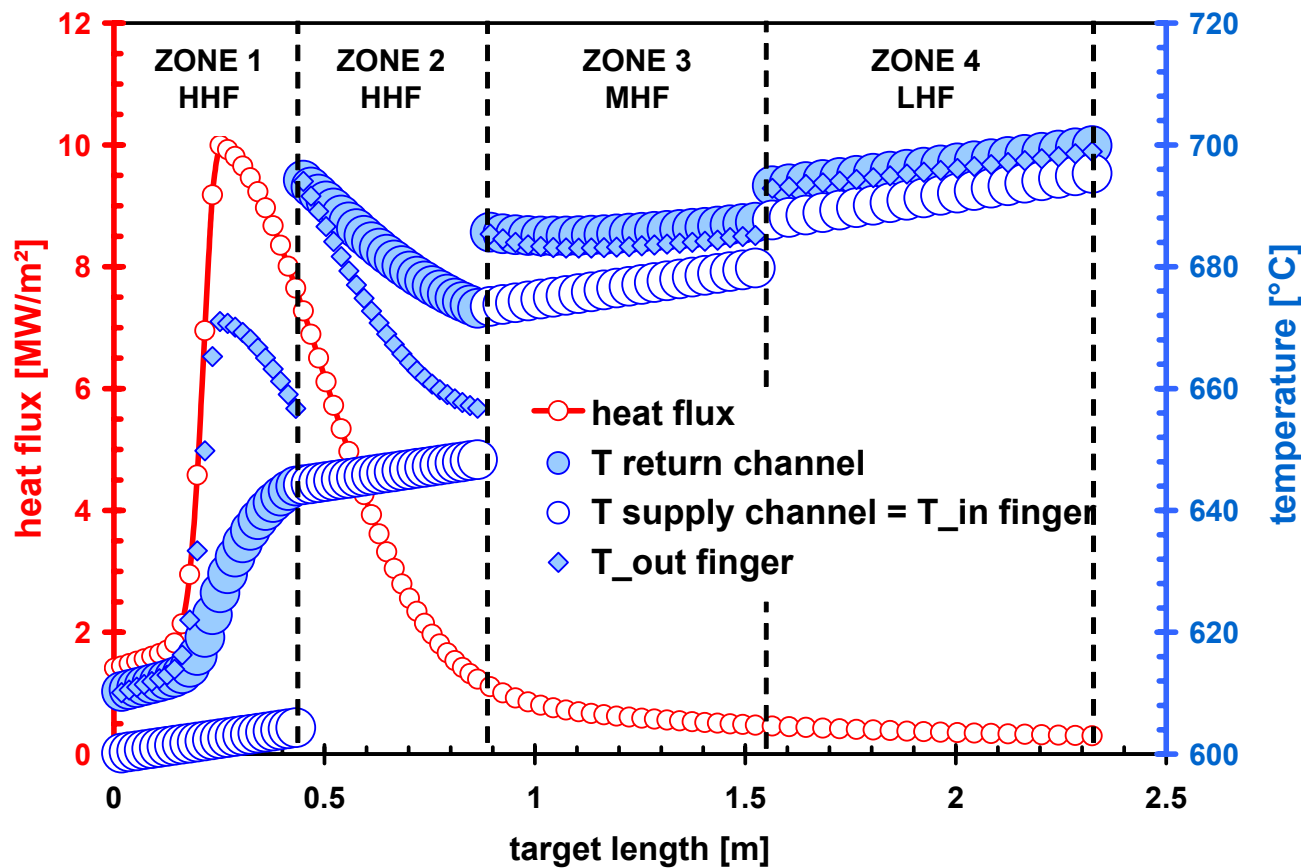
divertor power load
685 MW
8 % neutron power
2 % surface power

max. 80 % of
surface power
on outboard
→ 230 MW





LAYOUT EXAMPLE TOKAMAK: thermo-hydraulic layout $\Delta T = 100$ K





SUMMARY

Divertor Design Concept for ARIES CS has been developed

- New T-tube heat exchanger design
- New manifold concept for rel. high neutron load (vol. heating)
- Slot based jet impingement adapted
- Good therohydraulic performance (Said & Regina FZK)
- Thermal stress within design limits for 10 MW/m²
- Different Integration concepts can be chosen
- Detailed target layout possible, when distribution data available