

Progress of the EU Power Plant Conceptual Study (PPCS)

US/Japan Workshop on Power Plant Studies and Related Advanced Technologies With EU Participation

San Diego, 6-7 April 2002

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Outline of Presentation

⇒ PPCS Framework & Objectives

⇒ Status of PPCS Activities

- Requirements
- Design Studies
- Physics & Basic Parameters
- Divertors
- Maintenance
- Balance of Plant

⇒ Conclusions

PPCS Framework

- ⇒ PPCS (2000-2002): to establish coherence and priorities within the EU fusion programme
- ⇒ Stages 1 and 2 between Jan. 2000 and Apr. 2001 (preparatory work)
- ⇒ Stage 3 effective start in July 2001
 - ◆ Four models considered (A, B, C and D)
 - ◆ Point design for each model
 - ◆ Safety, environmental and economic assessments of each model

Why a PPCS in Europe?

⇒ Technical questions to be answered (raised during discussion on EU ITER site)

- ◆ What is a fusion reactor?
- ◆ How far are we from a fusion reactor?
- ◆ How attractive is a fusion reactor?
- ◆ Is the EU programme really reactor oriented?

2 key references: [1] R. Toschi et al, “How far is a fusion reactor from an experimental reactor”, SOFT 21 (2000); [2] K. Lackner et al., “Long-term fusion strategy in Europe”, ICFRM 10 (2001).

Basic Requirements (1/2)

⇒ Safety / Environment

- no need for emergency evacuation plan
- no active systems for safe shut-down

⇒ Operation

- steady state, >1 GWe, base load
- lifetime 40 yrs. (ext. to 60 yrs.)
- availability 75 ÷ 80 %, only a few unplanned shut-downs/year

(Requirements provided by Industry and Utilities)

Basic Requirements (2/2)

⇒ Economics

- public acceptance could be more important than economics
- economic comparison among equally acceptable energy sources (e.g. including externalities)
- licensing/regulation requirements strongly reduced vs. fission

⇒ Construction time ≤ 5 years

(Requirements provided by Industry and Utilities)

PPCS Design Studies (1/3)

- ⇒ Starting point: ITER, EU long term R&D programme, Safety & Socio-Economic studies

- ⇒ Main Effort: consistency of design
 - ◆ Physics assumptions
 - ◆ Divertor concepts
 - ◆ Maintenance
 - ◆ Balance of Plant

PPCS Design Studies (2/3)

⇒ 2 models with “limited” extrapolations

- Model A: WCLL (water cooled lithium lead)
- Model B: HCPB (helium cooled pebble bed)
- Divertor coolant = blanket coolant
- Normalised gross electric power output of 1.5 GWe
(turbogenerators power output less pumping power)

PPCS Design Studies (3/3)

⇒ 2 more “advanced” models

- Model C: DC (dual coolant: helium and lithium lead)
- Model D: SC (self cooled: lithium lead)
- Divertor coolant = blanket coolant (C: He, D: Li=Pb)
- Normalised gross electric **power output** (1.5 GWe)

Physics (1/2)

⇒ Assumptions for models A and B: ITER goals achieved, use “ITER physics rules”

⇒ $n/n_{GW} < 1.2$, $H_H < 1.2$, $\beta_N < 3.5$, first stability

⇒ Size driven by the divertor power handling capability (A: 15 MW/m²; B: 10 MW/m²)

⇒ Thermal efficiency of power conversion system (A: 31%, B: 40.5%)

Physics (2/2)

⇒ Goal for models C and D : scenario requiring less re-circulating power for CD without excessive loads on the FW

- High β and high confinement
- MHD stabilisation by strong plasma shaping
- High bootstrap current fraction
- Low divertor power loads and low Z_{eff}

⇒ divertor loads 10 MW/m² for C and 5 MW/m² for D, nuclear loads < 2.5 MW/m²

	Model A	Model B	Model C	Model D
Basic Parameters				
Major Radius (m)	9.8	8.7	7.5	6.1
Aspect Ratio	3.0	3.0	3.0	3.0
Plasma Current (MA)	33.5	28.1	20.1	14.1
Toroidal Field on axis (t)	7.3	6.9	6.4	5.6
TF on TF Coil Conductor (T)	12.9	13.1	13.6	13.4
Elongation (95% and separatrix)	1.7; 1.9	1.7; 1.9	1.9; 2.1	1.9; 2.1
Triangularity (95 % and separatrix)	0.27; 0.4	0.27; 0.4	0.47; 0.7	0.47; 0.7
Q	21	15	30	35
Physics Parameters				
H _H (IPB98y2)	1.2	1.2	1.3	1.2
n/ n _G	1.2	1.2	1.5	1.5
β _N (thermal); β(th ermal)	3.4; 4.8 %	3.3; 4.6 %	4.0; 5.0 %	4.5; 5.6 %
Bootstrap Fraction	0.36	0.36	0.69	0.76
q ₉₅	3	3	4.5	4.5
Z _{eff}	2.6	2.7	2.5	1.6
Engineering Parameters				
Fusion Power (GW)	5.5	3.4	3.4	2.5
P _{add} (MW)	265	234	112	71
Avg. Neutron Wall Load (MWm ⁻²)	2.3	1.8	2.2	2.4
Max. Divertor Heat Load (M Wm ⁻²)	15	10	10	5
Net Reactor Efficiency	27%	43%	44%	61%

Water Cooled Divertor (1/3)

⇒ ITER divertor

- ♦ Cu pipe armoured by CfC monoblock structure (lower region), W tiles in upper, baffle region
- ♦ Water temperature $< 200^{\circ}\text{C}$

⇒ However

- ♦ 18% of fusion power goes to the divertor
- ♦ If steel instead of Cu, max. heat load $< 7 \text{ MW/m}^2$

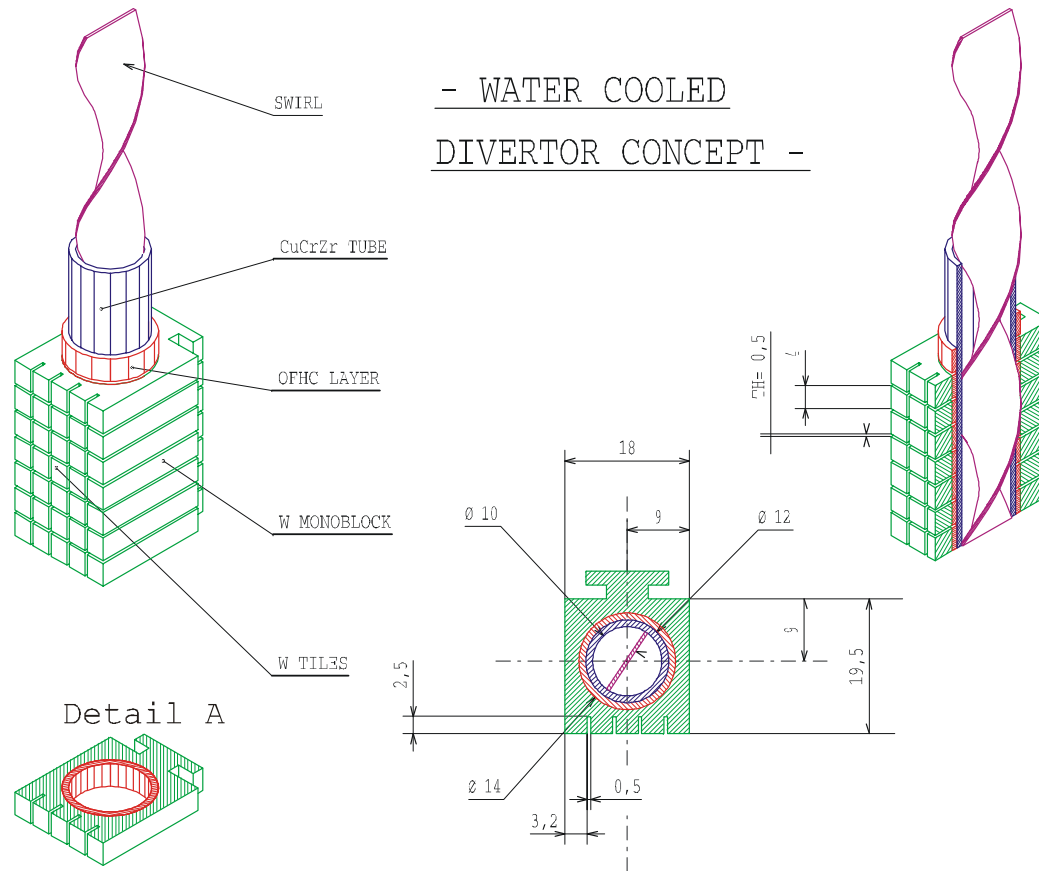
Water Cooled Divertor (2/3)

⇒ PPCS Model A divertor

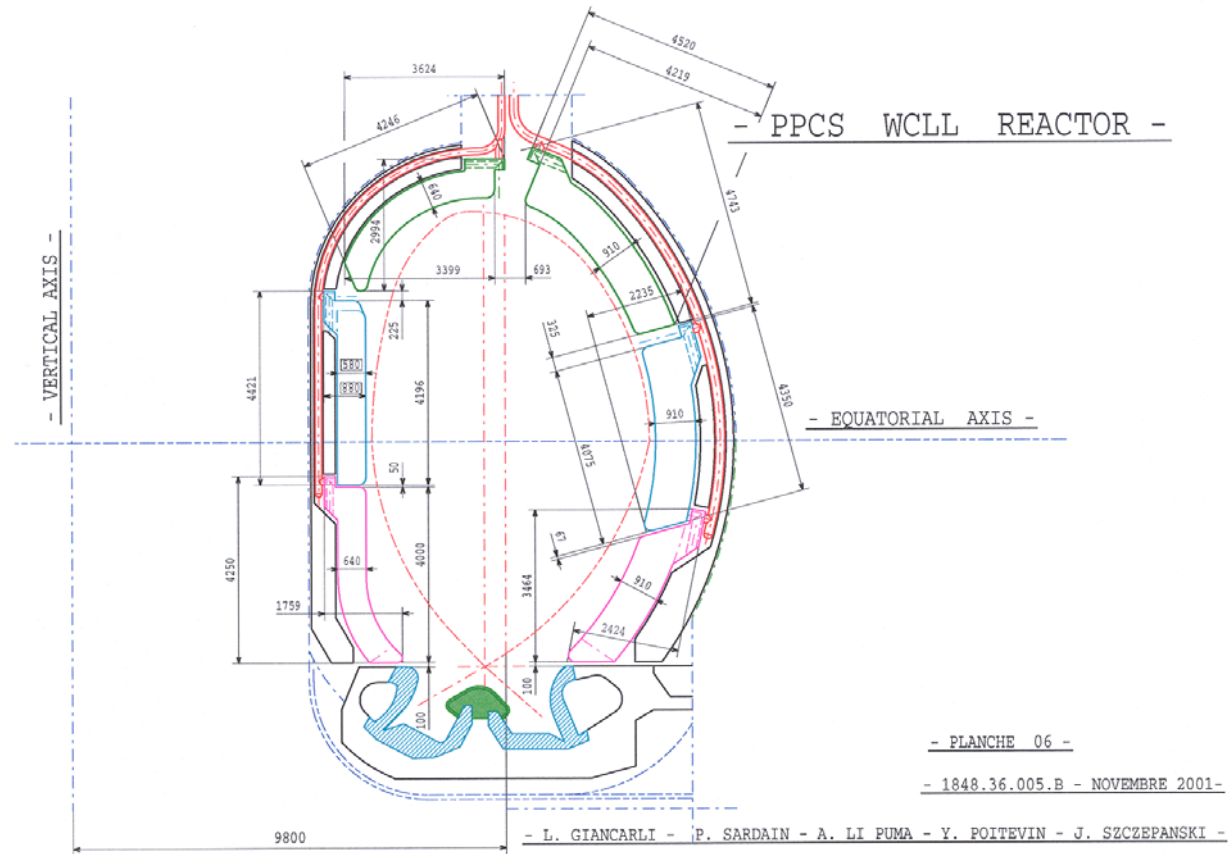
- ◆ Complete ELM suppression in H-mode
- ◆ Maximum heat-load (steady state) 15 MW/m²
- ◆ CuCrZr as heat sink, temp. <400°C, therefore water temp. <170°C
- ◆ W armour (low sputter yield, low tr. retention)
- ◆ Main issue: W embrittlement under neutron irradiation below 500°C

⇒ R&D required to validate concept

Water Cooled Divertor (3/3)



Cross-section of model A



Helium Cooled Divertor (1/3)

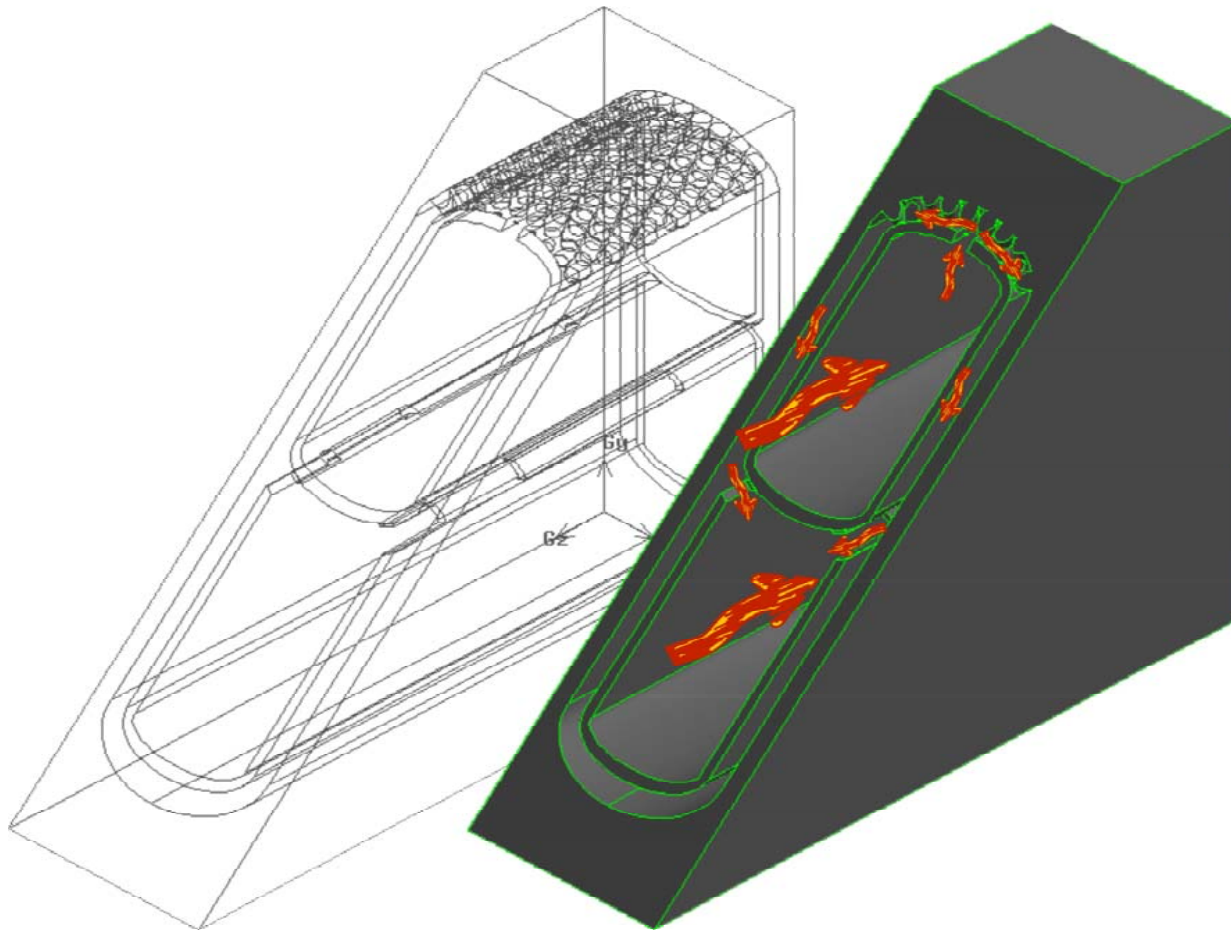
⇒ Why a He-cooled divertor?

- ◆ No risk of (accidental) Be-water reaction
- ◆ High grade heat
- ◆ Simplify BoP

⇒ Design issue: 10 MW/m²

- ◆ W-alloy (Mo alternative) structure and armour
- ◆ Very high, local He speed (200 m/s)
- ◆ equivalent heat transfer coefficient, i.e. related to the plain channel surface, of 48 kW/m²K

Helium Cooled Divertor (2/3)



Helium Cooled Divertor (3/3)

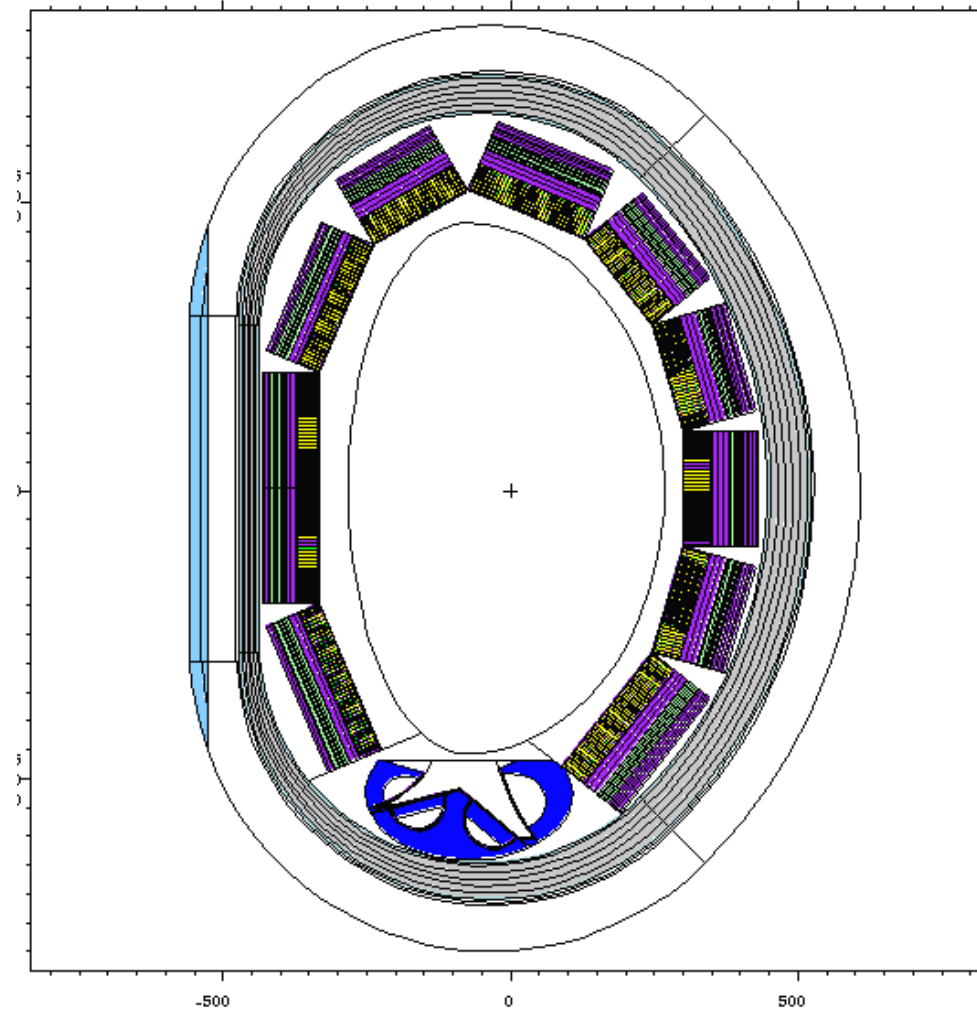
⇒ Temperatures

- He inlet temperature: 640°C, temp. gain: 100K.
- Temperature in structure 930°C - 1160°C
- Maximum temp. for 3.5 mm-thick tile: 1530°C

⇒ Von Mises Stresses below 400 MPa

⇒ Design & R&D required to validate concept

Cross-section of model B



Maintenance (1/3)

⇒ Two concepts assessed:

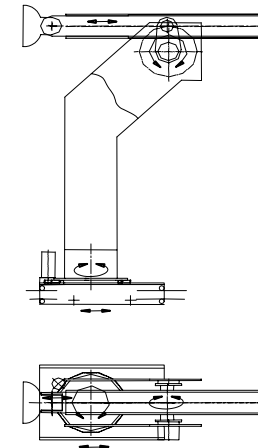
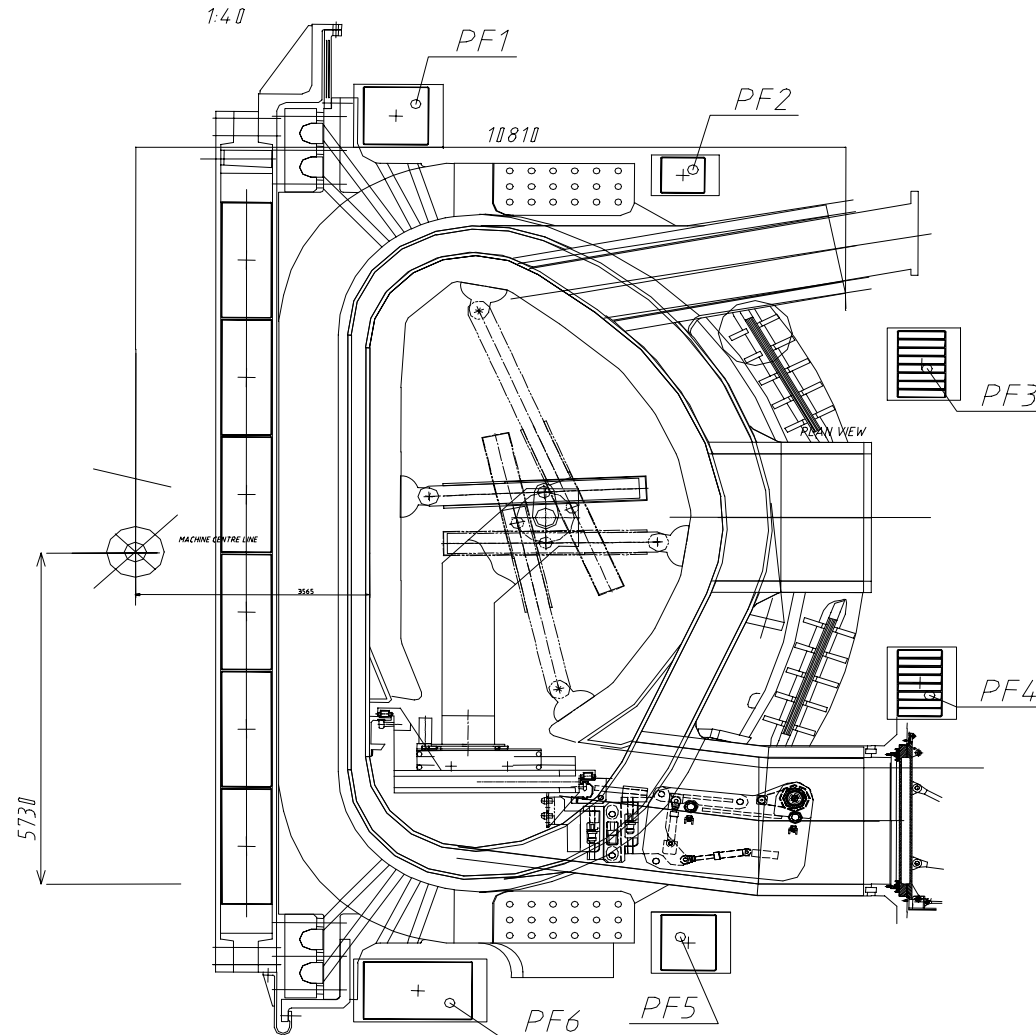
- ◆ “Large Modules”, i.e. ≈ 180 modules (ITER 1998: 720, ITER 2001:440)
- ◆ “Large Sectors”, as per ARIES-RS

⇒ Availability (lifetime: 5 FPY blanket; 2 FPY divertor and sectors)

- ◆ LM $\Rightarrow$ 70/75%, LS $\Rightarrow$ $< 80\%$

Maintenance (2/3)

Large Module Handling



Maintenance (3/3)

- ⇒ **Large Sectors**: several engineering issues TBD, >20% higher capital investment
- ⇒ **Large Module** Handling Concept confirmed as reference for models A and B
- ⇒ For models C and D **Large Module or Vertical "bananas" Segments** being considered

Balance of Plant (1/2)

- ⇒ **BoP**: approx. **40%** of capital investment, main cause of fission plants **unscheduled outages** in the 80's
- ⇒ **Model A**: blanket cooling water at **PWR conditions** (divertor water: 170°C)
- ⇒ **Model B**: limited experience; no feasibility issue but **“infant” mortality** ought to be considered (increase cost and/or reduce availability)

Balance of Plant (2/2)

⇒ Power Conversion

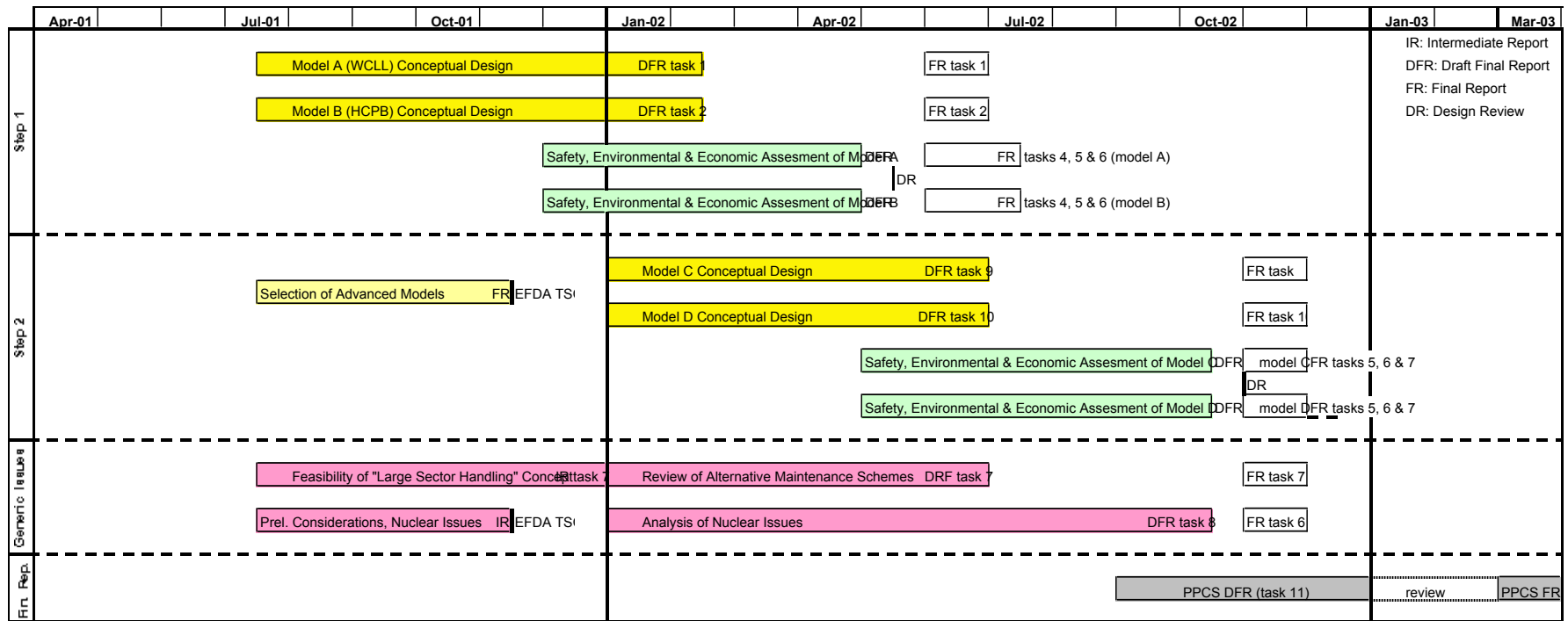
- ◆ steam turbine cycle for A, B, possible for C
- ◆ direct gas turbine cycle for D, possible for C

⇒ Net Reactor Efficiency:

(fusion power)/(electricity to grid)

- ◆ A: 27%, B: 43%, C: 44%, D :61%

PPCS St. 3 Original Schedule



Conclusions (1/2)

- ⇒ Design of **conservative models** being finalised, assessments in progress
- ⇒ Design of **advanced models** started
- ⇒ **Main emphasis**: physics, divertor, maintenance and BoP
- ⇒ PPCS **schedule** roughly on time (FR in 03.03)

Conclusions (2/2)

⇒ R&D requirements

- ◆ Blankets & materials: continue, assess HCLL concept
- ◆ Divertor: critical, to be defined and implemented
- ◆ Physics: continue work on advanced scenarios
- ◆ CD: highly efficient (70%) systems required
- ◆ Maintenance: validate “large module” concept

PPCS Papers at ISFNT-6

⇒ Invited NSD INV. 1 (Parallel Session #5)

- ◆ “The EU Power Plant Conceptual Study” by G. Marbach, I. Cook and D. Maisonnier

⇒ Poster FWC POS. 16 (Poster Session #4)

- ◆ “Optimisation of a Water-Cooled Divertor for the European Power Plant Conceptual Study” by A. Li Puma et al.

⇒ Poster FWC POS. 23 (Poster Session #4)

- ◆ “Gas-cooled High Performance Divertor for a Power Plant” by S. Hermesmeyer and S. Malang