

Power Core Safety Analysis Results

**Paul Humrickhouse
Brad Merrill**

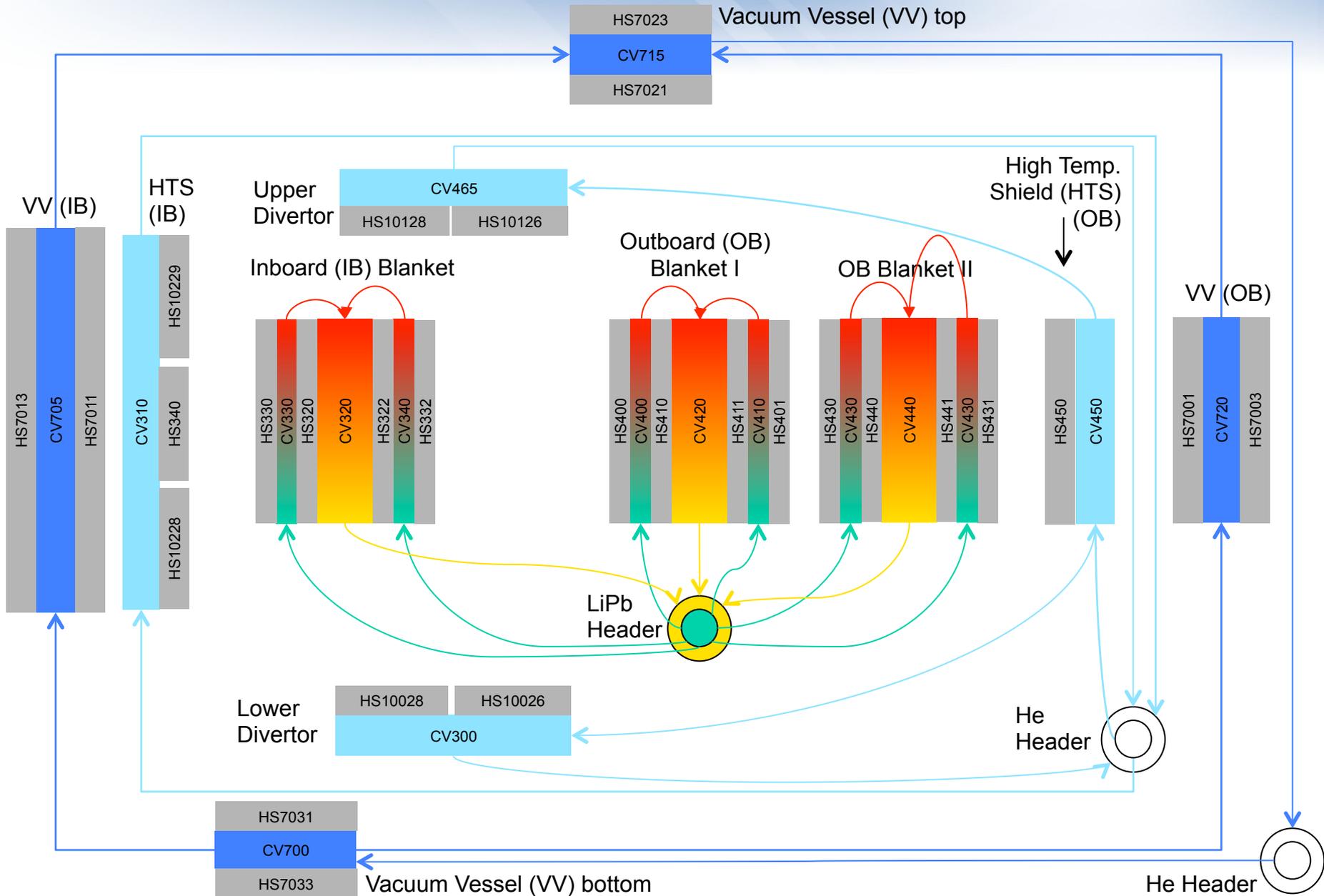
INL Fusion Safety Program

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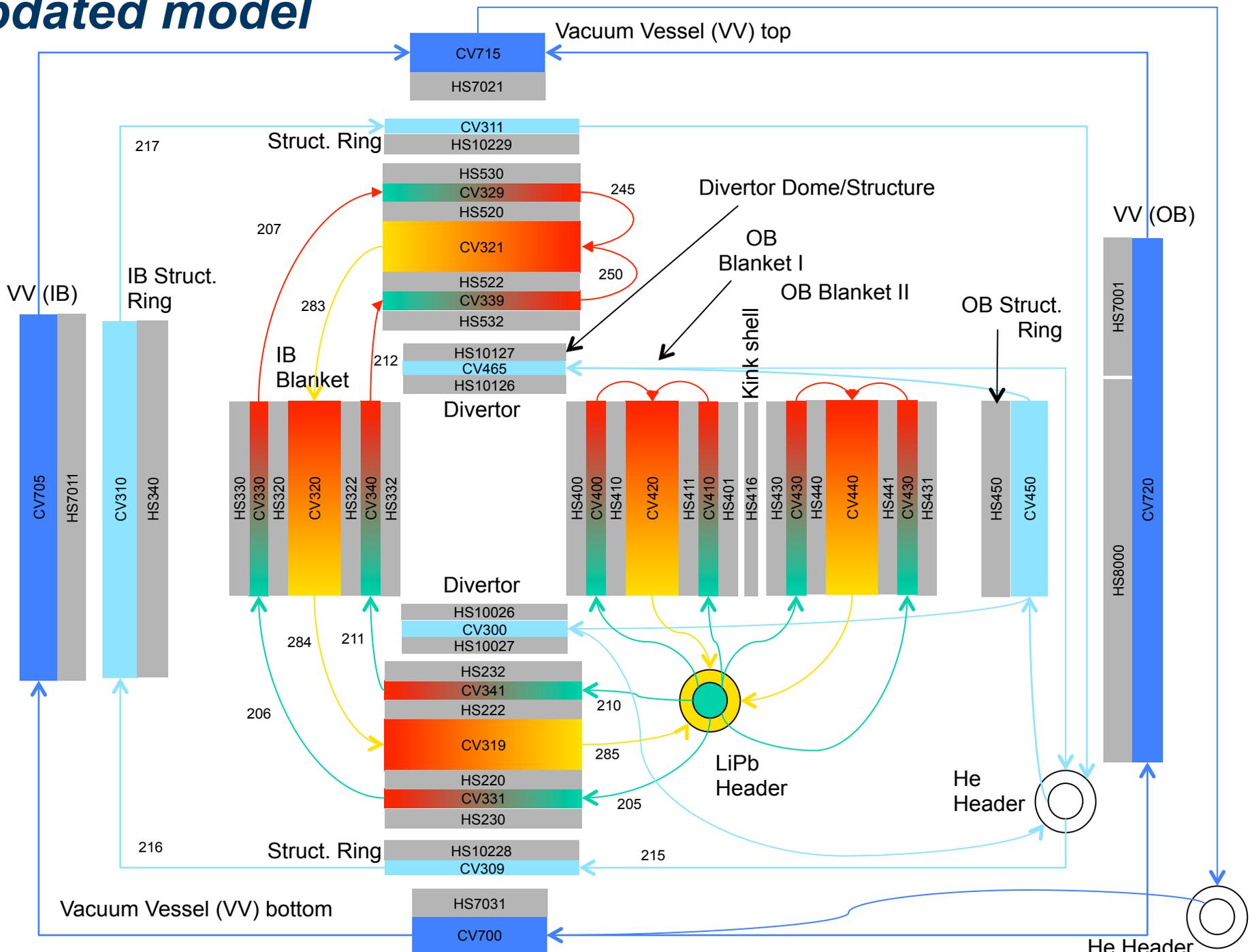


Updates to MELCOR model since February

- A substantial revision of the base MELCOR model has been undertaken since February to address ACT-1 design changes
 - Final radial build updated
 - Shield plugs added
 - Upper and lower blanket structures and volumes added
 - Kink shell added
 - Divertor dome/steel structure added
 - Modification of decay heats (from Laila) completed
 - Revision of all nuclear heats (Laila's Sept. presentation)
 - He cooling system resized
- Nuclear and decay heats are resulting in higher than desired temperatures



Updated model



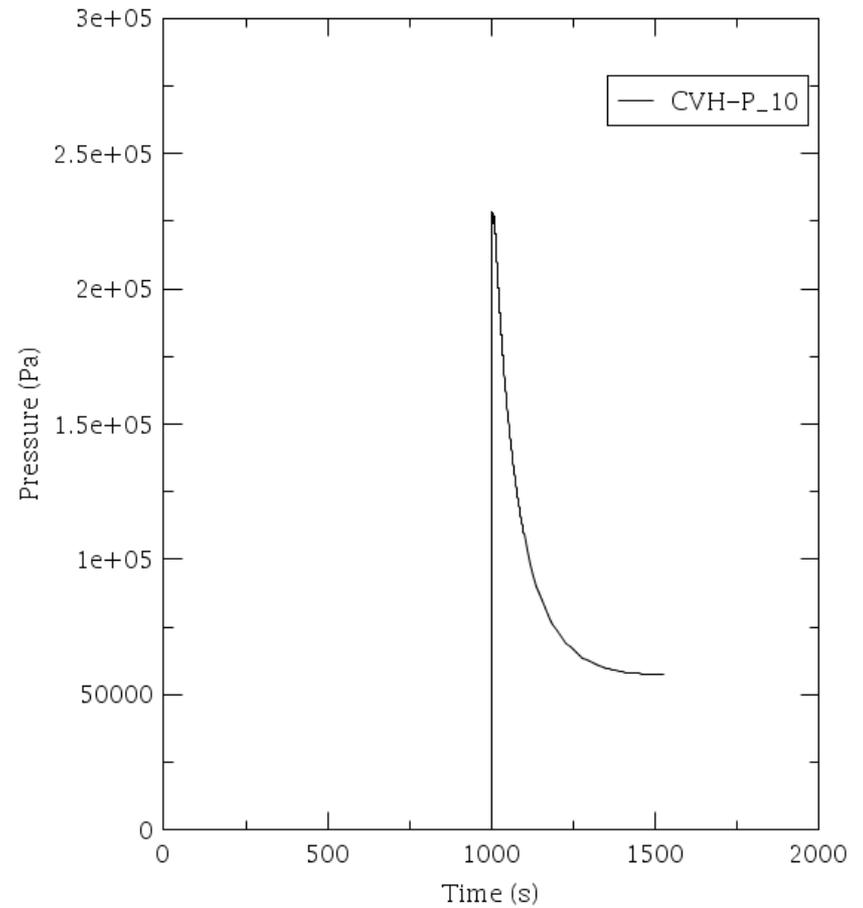
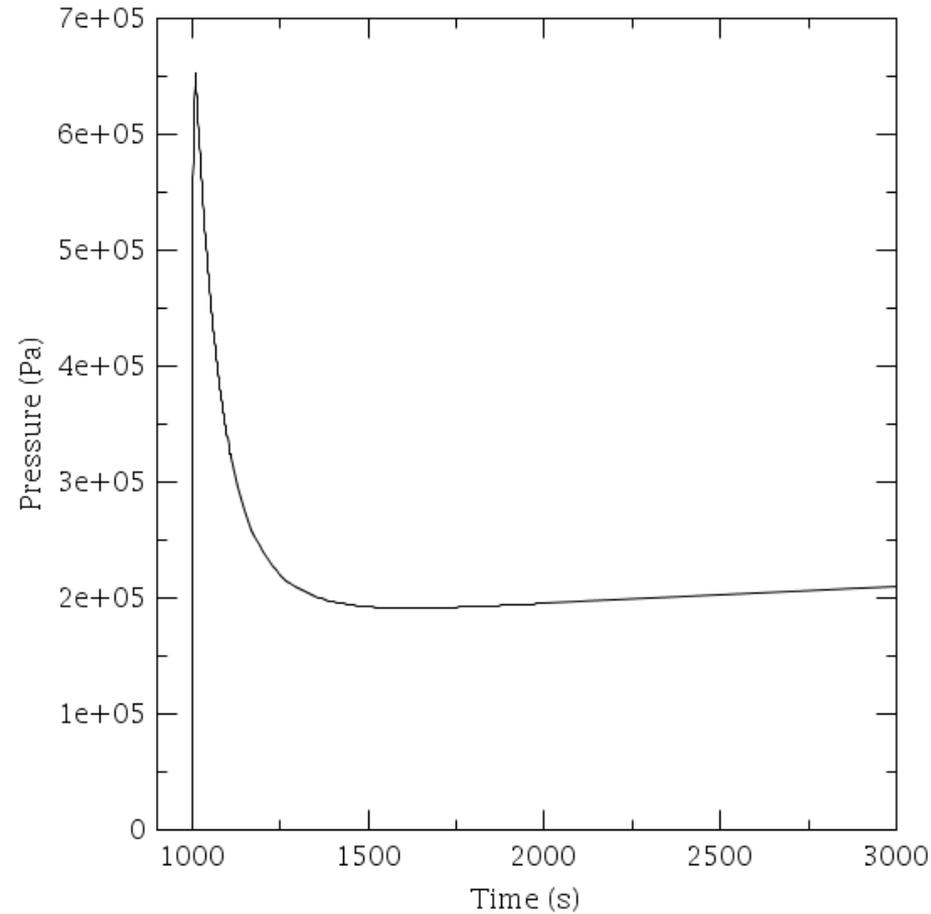
He LOCA Analysis

- A Helium (Divertor/steel ring loop) ex-vessel LOCA has been modeled
 - Double guillotine break in outlet pipe
- Primary concern: overpressure in cryostat leading to release of radioactive material
- ARIES-CS guidance: 3×10^5 Pa is pressure limit for the cryostat
- Pressure was too high ($\sim 6.5 \times 10^5$ Pa) based on previous model
 - Ex-vessel coolant volumes resized such that there is a ratio of 2:1 ex-vessel:in-vessel He

He LOCA Analysis

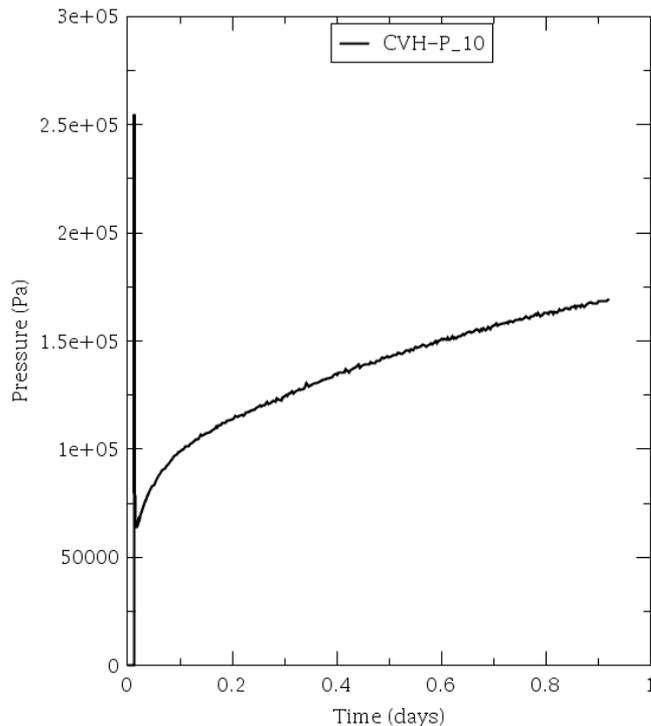
- Previous model

Current model

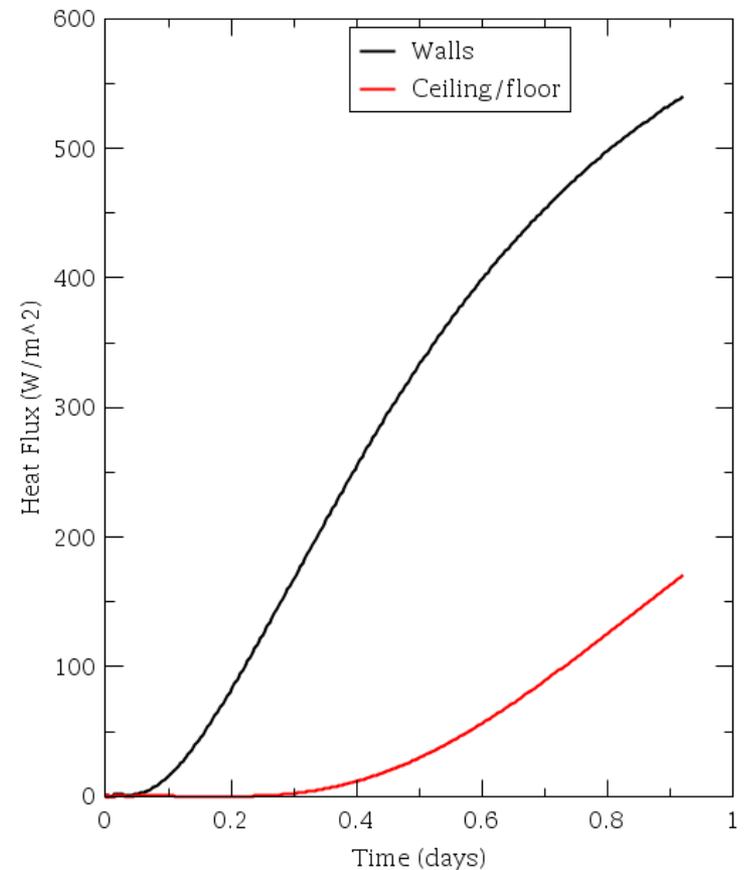


He LOCA Analysis

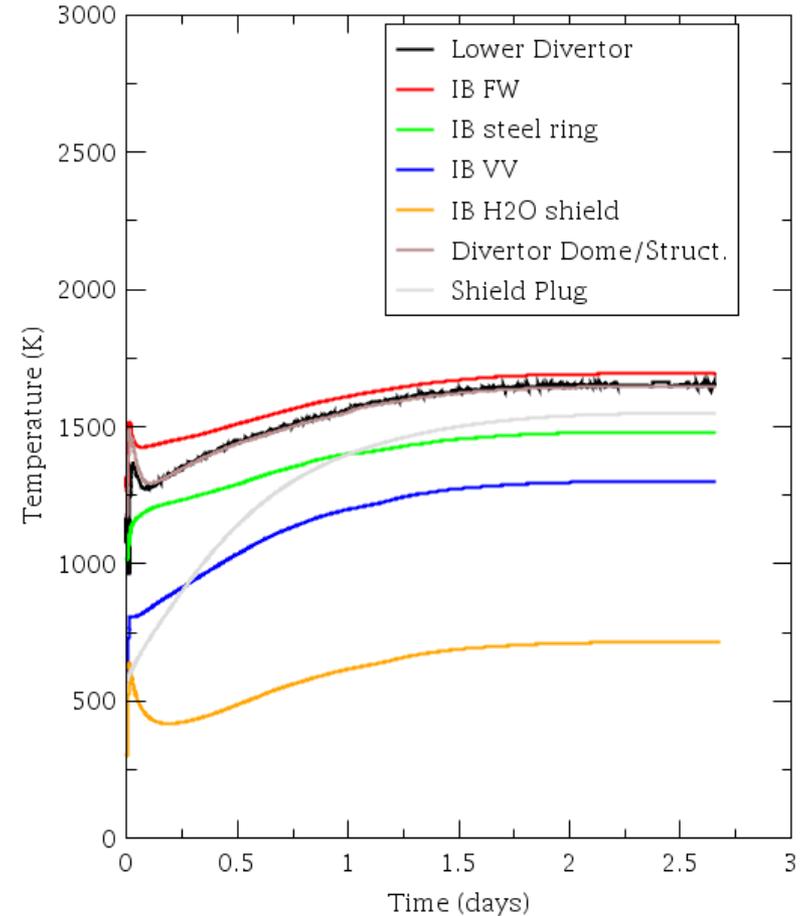
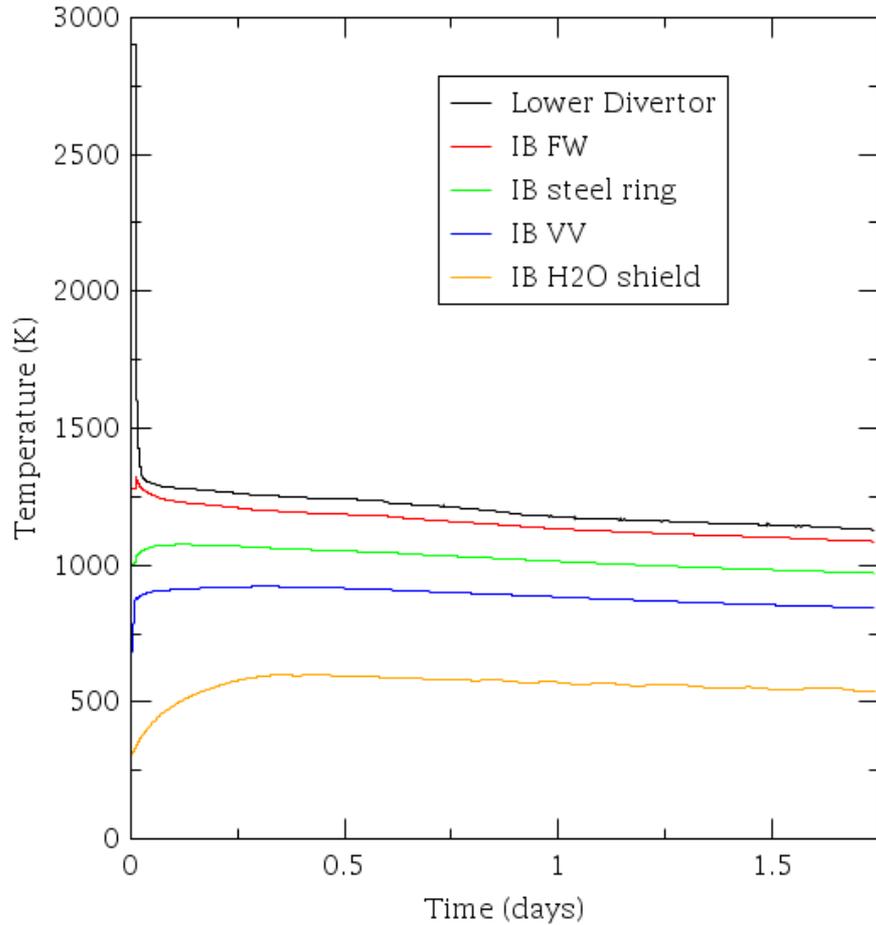
- At longer times, the cryostat pressure builds and heat is transferred through the walls to the environment



Heat transfer from Cryostat to vault



Structure temperatures during LOFA



Temperature and Heat Transfer issues

- New nuclear and decay heats have led to high structure temperatures
 - The preceding were for a LOFA
 - They will be worse in the combined LOFA/water LOCA
- In some cases, these are probably resulting from necessary simplifications in MELCOR
 - Divertor is just a slab with default HT correlations
 - Divertor steel structure is a relatively thick slab with cooling on one side
 - Shield plugs are thick (~45 cm) and cooled on one side using the external surface area; presumably there is more heat transfer area in the actual design
 - IB WC shield cannot be maintained at ~300 K with natural convection if the nuclear heating is ~50 MW
 - Flow rates may need to increase
- Any input on the above appreciated

LOFA+LOCA analysis

- LOFA plus water LOCA case is running (a MELCOR bug has been identified by this case and is being resolved)
- Preliminary results show some interesting natural circulation phenomena

